

LICENSEE EVENT REPORT

CONTROL BLOCK:

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(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME:

01	N	Y	I	P	S	3
----	---	---	---	---	---	---

 LICENSE NUMBER:

0	0	-	0	0	0	0	0	-	0	0
---	---	---	---	---	---	---	---	---	---	---

 LICENSE TYPE:

4	1	1	1	1
---	---	---	---	---

 EVENT TYPE:

0	3
---	---

REPORT TYPE:

L

 REPORT SOURCE:

L

 DOCKET NUMBER:

0	5	0	-	0	2	8	6
---	---	---	---	---	---	---	---

 EVENT DATE:

0	6	0	8	7	6
---	---	---	---	---	---

 REPORT DATE:

0	7	0	8	7	6
---	---	---	---	---	---

EVENT DESCRIPTION

02 | _____ | 80
03 | See | 80
04 | Attached | 80
05 | Sheet | 80
06 | _____ | 80

SYSTEM CODE:

R	B
---	---

 CAUSE CODE:

D

 COMPONENT CODE:

C	K	T	B	R	K
---	---	---	---	---	---

 PRIME COMPONENT SUPPLIER:

N

 COMPONENT MANUFACTURER:

X	9	9	9
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 VIOLATION:

N

CAUSE DESCRIPTION

09 | Control rod P-6 dropped because a fuse which shared the negative lead | 80
09 | to the movable gripper of both rod N-7 and P-6 was pulled. The | 80
10 | applicable procedure did not specify the correct fuse to be removed. | 80

FACILITY STATUS:

B

 % POWER:

0	7	4
---	---	---

 OTHER STATUS:

NA

 METHOD OF DISCOVERY:

A

 DISCOVERY DESCRIPTION:

NA

FORM OF ACTIVITY RELEASED:

Z

 CONTENT OF RELEASE:

Z

 AMOUNT OF ACTIVITY:

NA

 LOCATION OF RELEASE:

NA

PERSONNEL EXPOSURES

13 | NUMBER:

0	0	0
---	---	---

 TYPE:

Z

 DESCRIPTION:

NA

 | 80

PERSONNEL INJURIES

14 | NUMBER:

0	0	0
---	---	---

 DESCRIPTION:

NA

 | 80

PROBABLE CONSEQUENCES

15 | NA | 80

LOSS OR DAMAGE TO FACILITY

16 | TYPE:

Z

 DESCRIPTION:

NA

 | 80

PUBLICITY

17 | NA | 80

8111070538 760708
PDR ADOCK 05000286
S PDR

ADDITIONAL FACTORS

18 | (Cause Desc. Cont.) Procedure INT-TP-9.6 was changed as necessary to | 80
19 | clarify the steps dealing with the removal of fuses. | 80

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EVENT DESCRIPTION

While performing Dropped Rod Test INT-TP-9.6, control rod N-7 was dropped as required by the test procedure. Approximately 15 seconds later, control rod P-6 dropped. The dropping of control rod N-7 initiated a turbine runback from 74% to 49% as required. When control bank D inserted to maintain programmed T_{avg} , control rod P-6 dropped. The maximum radial tilt calculated following rod withdrawal was 6.7%. Since the high flux trip setpoint was set at 85% prior to the start of the test, no additional adjustment was required. Analysis of core peaking factors by Nuclear Engineering showed that minimum DNBR requirements were not violated. [R.O.-76-3-19(B)].