Consolidated Edison Company of New York, Inc. 4 Irving Place, New York, N Y 10003 Telephone (212) 460-3819

DECIT

UDA

October 14, 1976

Re: Indian Point Unitation.

Docket No. 50-28671/

R.O. -76-3-30(A)

OCT 1 81975

Mr. James P. O'Reilly, Director Office of Inspection and Enforcement Region 1 U.S. Nuclear Regulatory Commission King of Prussia, PA 19406

Dear Mr. O'Reilly:

In accordance with the requirements of the Technical Specifications to Facility Operating License DPR-64 the attached report of Reportable Occurrence R.O. -76-3-36(A) is submitted. This report fulfills the requirement for a written report within 14 days of a Reportable Occurrence and is in accordance with the format set forth in Regulatory Guide 1.16, Revision 4.

Three copies of this letter and the attachment are enclosed as required.

Very truly yours,

William J. Cahill, Jr.

Vice President

Enc: PP/mmg

Copy to

Director of Nuclear Reactor Regulation
ATTN: Dr. Ernst Volgenau, Director (40 copies)
Office of Inspection and Enforcement

U.S. Nuclear Regulatory Commission

Washington, D.C. 20555

Director of Nuclear Reactor Regulation
ATTN: Mr. William G. McDonald, Director (3 copies)
Office of Management Information and
Program Control

U.S. Nuclear Regulatory Commission
Washington D.C. 20555

Washington, D.C. 20555
Mr. George T. Berry

Mr. George T. Berry General Manager and Chief Engineer Power Authority of the State of New York 10 Columbus Circle

Yew York, N.Y. 10019

## Event Description

Following venting of the reactor coolant system preparatory to plant heatup, a pressure transient to approximately 2250 psig was experienced. At the time of the occurrence, the system was being operated in the solid mode at a temperature of approximately 180°F. Analysis of this event by Con Edison Engineering and Westinghouse personnel in light of the basis of Technical Specification 3.1.B.1 and a metallurgical evaluation of the vessel were conducted. It was concluded that the integrity of the reactor vessel had not been impaired.

(R.O. -76-3-36(A))

## Cause Description

Overpressurization of the RCS was the result of a reduction in letdown flow caused by a probable inadvertent closure of one or both
RHR isolation valves (Plant Designation MOV-730 and MOV-731, 14"
Copes Vulcan Gate Valves, Mfgr. Assembly Dwg. E-1-133402). A
check of the control circuits for these valves is in progress. To
assist the operator in a more timely identification of potential
overpressurization problems, an Engineering Service Request has
been initiated to provide narrow range RCS pressure instrumentation
and variable setpoint alarms for operation during cold shutdown
conditions. In addition, Consolidated Edison is participating
in a program, along with Westinghouse and other utilities, intended to provide solutions to the general problem of reactor coolant
system pressure transients.