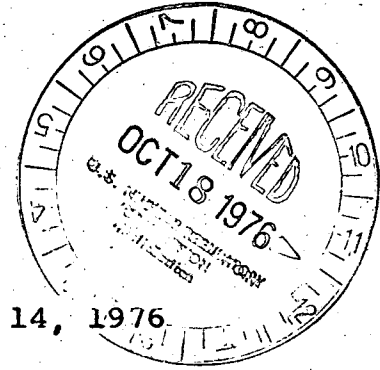


William J. Cahill, Jr.
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, N Y 10003
Telephone (212) 460-3819

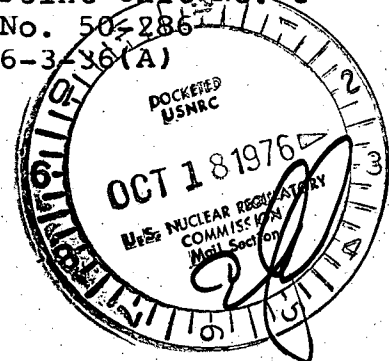
REGISTERED

COPY



October 14, 1976

Re: Indian Point Unit No. 3
Docket No. 50-286
R.O. -76-3-36(A)



Mr. James P. O'Reilly, Director
Office of Inspection and Enforcement
Region 1
U.S. Nuclear Regulatory Commission
King of Prussia, PA 19406

Dear Mr. O'Reilly:

In accordance with the requirements of the Technical Specifications to Facility Operating License DPR-64 the attached report of Reportable Occurrence R.O. -76-3-36(A) is submitted. This report fulfills the requirement for a written report within 14 days of a Reportable Occurrence and is in accordance with the format set forth in Regulatory Guide 1.16, Revision 4.

Three copies of this letter and the attachment are enclosed as required.

Very truly yours,

William J. Cahill, Jr.
William J. Cahill, Jr.
Vice President

Enc:
PP/mmg

Copy to : Director of Nuclear Reactor Regulation
ATTN: Dr. Ernst Volgenau, Director (40 copies)
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director of Nuclear Reactor Regulation
ATTN: Mr. William G. McDonald, Director (3 copies)
Office of Management Information and
Program Control
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. George T. Berry
General Manager and Chief Engineer
Power Authority of the State of New York
10 Columbus Circle
New York, N.Y. 10019

8111070457 761014
PDR ADOCK 05000286
S PDR

10508

LICENSEE EVENT REPORT

R.O. -76-3-36(A)

CONTROL BLOCK:

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(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME						LICENSE NUMBER										LICENSE TYPE				EVENT TYPE					
01	N	Y	I	P	S	3	0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1	0	1	
7	8	9				14	15											25	26				30	31	32
01 CONT		CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER						EVENT DATE				REPORT DATE									
01	CONT			T	L	0	5	0	-	0	2	8	6	0	9	3	0	7	6	1	0	1	4	7	6
7	8	57	58	59	60	61							68	69					74	75					80

EVENT DESCRIPTION

02																								80
03																								80
04	SEE ATTACHED SHEET																							80
05																								80
06																								80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION		
07	C	B	E	V	A	L	V	E	X	N	C	6	3	5	Y	
7	8	9	10	11	12					17	43	44			47	48

CAUSE DESCRIPTION

08																								80
09	SEE ATTACHED SHEET																							80
10																								80

FACILITY STATUS		% POWER			OTHER STATUS			METHOD OF DISCOVERY		DISCOVERY DESCRIPTION															
11	G	U	0	0	0	NA	A	Control Room Instrumentation																	
7	8	9	10	11	12	13	44	45	46																80
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE																	
12	Z	Z	NA	NA				NA																	
7	8	9	10	11																					80

PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION	
13	0	0	0	Z	NA
7	8	9	11	12	13

PERSONNEL INJURIES

NUMBER		DESCRIPTION		
14	0	0	0	NA
7	8	9	11	12

PROBABLE CONSEQUENCES

15	NA																							80
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LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION	
16	Z	NA	
7	8	9	10

PUBLICITY

17	NA																							80
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ADDITIONAL FACTORS

18	NA																							80
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19																								80
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NAME: Austin J. Decker II

PHONE: 914-739-8823

Event Description

Following venting of the reactor coolant system preparatory to plant heatup, a pressure transient to approximately 2250 psig was experienced. At the time of the occurrence, the system was being operated in the solid mode at a temperature of approximately 180°F. Analysis of this event by Con Edison Engineering and Westinghouse personnel in light of the basis of Technical Specification 3.1.B.1 and a metallurgical evaluation of the vessel were conducted. It was concluded that the integrity of the reactor vessel had not been impaired.

(R.O. -76-3-36(A))

Cause Description

Overpressurization of the RCS was the result of a reduction in let-down flow caused by a probable inadvertent closure of one or both RHR isolation valves (Plant Designation MOV-730 and MOV-731, 14" Copes Vulcan Gate Valves, Mfgr. Assembly Dwg. E-1-133402). A check of the control circuits for these valves is in progress. To assist the operator in a more timely identification of potential overpressurization problems, an Engineering Service Request has been initiated to provide narrow range RCS pressure instrumentation and variable setpoint alarms for operation during cold shutdown conditions. In addition, Consolidated Edison is participating in a program, along with Westinghouse and other utilities, intended to provide solutions to the general problem of reactor coolant system pressure transients.