

William J. Cahill, Jr.  
Vice President

# Regulatory Docket File

Consolidated Edison Company of New York, Inc.  
4 Irving Place, New York, N Y 10003  
Telephone (212) 460-3819

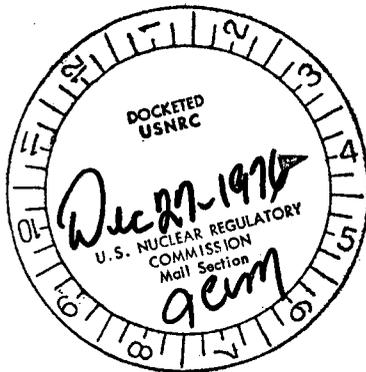
December 15, 1976

Re: Indian Point Unit No. 3  
Docket No. 50-286

Mr. James P. O'Reilly, Director  
Office of Inspection and Enforcement  
Region 1  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

Dear Mr. O'Reilly:

The attached report (TEIR-76-3-1) is being submitted for  
your information.



Very truly yours,

A handwritten signature in cursive script that reads "William J. Cahill, Jr.".

William J. Cahill, Jr.  
Vice President

PK/mmg

Copy to: Mr. George T. Berry  
General Manager and Chief Engineer  
Power Authority of the State of New York  
10 Columbus Circle  
New York, N.Y. 10019

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PDR ADDCK 05000286  
S PDR

13003

# LICENSEE EVENT REPORT

TEIR-76-3-1

CONTROL BLOCK: 

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(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME			LICENSE NUMBER												LICENSE TYPE				EVENT TYPE							
01	N	Y	I	P	S	3	0	0	-	0	0	0	0	d	-	0	0	4	1	1	1	1	9	9		
7	8	9	14	15	25	26	30	31	32	CATEGORY			REPORT TYPE	REPORT SOURCE	DOCKET NUMBER				EVENT DATE				REPORT DATE			
01	CONT																									
7	8	57	58	59	60	61	68	69	74	75	80															

### EVENT DESCRIPTION

02																								80
03																								80
04	SEE ATTACHED SHEET																							80
05																								80
06																								80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER				VIOLATION	
07	C	F	E	V	A	L	V	E	X	N	C	7	1	0	N
7	8	9	10	11	12	17	43	44	47	48					

### CAUSE DESCRIPTION

08	A two inch Crosby relief valve, Mfg. Assembly Dwg. No. H-51680,																							80
09	lifted at 430 psig. Pressure relief setting for this valve,																							80
10	plant designation RV-1836, is 600 psig.																							80

FACILITY STATUS		% POWER			OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION			
11	C	0	0	0	NA				A	Supervisory Panel Alarms				
7	8	9	10	12	13	44	45	46	80					
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE			AMOUNT OF ACTIVITY				LOCATION OF RELEASE					
12	Z	Z	NA				NA							
7	8	9	10	11	44	45	80							

### PERSONNEL EXPOSURES

NUMBER		TYPE		DESCRIPTION					
13	0	0	Z	NA					
7	8	9	11	12	13	80			

### PERSONNEL INJURIES

NUMBER		DESCRIPTION						
14	0	0	NA					
7	8	9	11	12	80			

### PROBABLE CONSEQUENCES

15	NA																							80
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### LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION					
16	Z	NA					
7	8	9	10	80			

### PUBLICITY

17	NA																							80
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### ADDITIONAL FACTORS

18	NA																							80
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19																								80
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NAME: Austin J. Decker II PHONE: 914-739-8823

FOR INFORMATION ONLY

During plant heatup, with a bubble in the pressurizer, RHR system in service, and the RHR pumps secured, pressurizer level began decreasing rapidly. All reactor coolant pumps were shut down and a second charging pump was started to stabilize pressurizer level. Concurrently with the pressurizer level decrease, a level increase and resultant pressure increase was noted in the pressurizer relief tank. In order to maintain a pressure of less than 30 psig in this tank, the dump valve to the containment sump was opened.

Upon investigation, it was found that RHR suction relief valve (plant designation RV-1836), which discharges to the pressurizer relief tank, had lifted. The discharge from the valve was terminated by manual closure. After conditions were stabilized, a reactor coolant pump was re-started and the RHR system isolated. The setting for this valve is 600 psig, however at no time during this occurrence did the RCS pressure exceed 430 psig.

During the next outage requiring the RHR system to be placed in service, the relief valve will be reset to actuate at its design set pressure. In addition, a replacement valve has been ordered which will be preset at the factory.

USNRC-REG. 1-

17 DEC 76 10: 04