

William J. Cahill, Jr.
Vice President

Regulatory

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April 9, 1973

Re Indian Point Unit No. 3
AEC Docket No. 50-286

Mr. Richard C. DeYoung
Assistant Director for
Pressurized Water Reactors
Directorate of Licensing
U. S. Atomic Energy Commission
Washington, D. C. 20545



Dear Mr. DeYoung :

Your letter dated March 22, 1973 requested modifications to certain portions of the Indian Point Unit No. 3 electrical and mechanical design and the submittal of a comprehensive Industrial Security Plan.

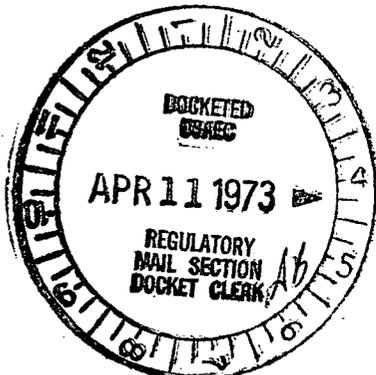
Con Edison's position with regard to the electrical and mechanical design items is delineated in Attachment A. The FSAR will be amended, where necessary, to update information affected by the modifications described in this letter.

Three (3) copies of a document entitled, "Indian Point Unit No. 3 Security Plan", have been filed as a proprietary submittal as part of Supplement 16 to the FSAR.

Very truly yours,

William J. Cahill, Jr.
Vice President

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ATTACHMENT A

Comments on Enclosure 1 of AEC Letter

1. Disconnection of Instrument Bus 33

Instrument Bus 33 will be fed by Battery 33 through a new inverter to provide an uninterruptible power supply.

2. Injection Line Flow Instrumentation

Calculations by Westinghouse indicate that ~ 525 gpm to the core is required to match boil-off at 1800 seconds (the earliest time at which recirculation could be initiated). Accordingly, a requirement of 600 gpm minimum flow rate has been specified (FSAR Page 6.3-16) to account for uncertainties in flow measurement and to provide margin.

The decision making process presented on FSAR Page 6.2-14 with regard to the flow to the Reactor Coolant System via the low head injection lines should be based on two of four flow meters, each reading ≥ 600 gpm. The rationale for this basis is as follows:

- a) two flow meters fail to a low reading (as a result of a single failure) - flow rate unknown, assume zero,
- b) one flow meter reads ≥ 600 gpm, but it is connected to spilling line; therefore, flow is ineffective,
- c) one flow meter reads ≥ 600 gpm, and thereby ensure that 600 gpm (or more) is delivered to the core.

No modifications to the present design are considered necessary. FSAR Pages 6.2-14 and Q7.14-1, and Tables 6.2-7(b) and 7.2.5 will be amended to reflect this.

Comments on Enclosure 2 of AEC Letter

1. Seismic Instrumentation

- a) Three Engdahl Enterprises Peak Shock Recorders (Model PSR 1200-H/V-11A) or an equivalent unit will be installed in a tri-axial arrangement on the Containment Base Mat.

This unit, with the tri-axial arrangement, will provide a plot of eleven points on the 2% damping curve for the vertical axis and two horizontal axes. The eleven points will be within the frequency range of 2-26 Hz.

In addition, a Peak Shock Annunciator (Model PSA 1575) will be installed in the Unit No. 3 Central Control Room. The PSA 1575 unit will give a visual warning that predetermined acceleration limits, making up the 2% damping response spectrum, have been exceeded at any or all of the eleven frequencies monitored.

- b) A Teledyne Peak Recording Accelerograph (Model PRA-100) or an equivalent unit, will be installed on one steam generator, on one Reactor Coolant Pump and on the pressurizer.

A plan for the utilization of the data obtained from the seismic instrumentation in the event of an earthquake will be developed.

2. Pre-Operational Piping Dynamic Effects Test Program

The response to Question 4.31 in the FSAR will be revised to indicate that during the normal course of the pre-operational test program specific attention will be directed at evaluating possible vibration problems during thyperformance of various transients. A list of these transient conditions will be included in the revised response.

If vibration problems are observed, further investigation will be performed to establish and correct any adverse conditions. Another test will be performed to demonstrate the adequacy of any corrective modifications.