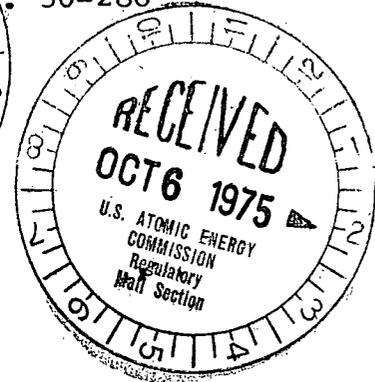
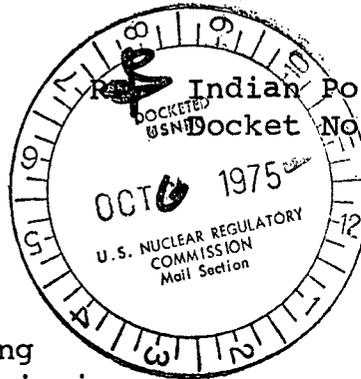


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Regulatory

File Cy.

September 30, 1975



Mr. D. B. Vassallo, Chief
Light Water Reactors
Project Branch 1-1
Division of Reactor Licensing
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Vassallo:

The information requested by Items 1 and 2 of your letter of July 22, 1975 were forwarded to you on July 30 and September 4, 1975 respectively.

Items 3 and 4 requested information on vessel support loads and stresses and plant conditions resulting from postulations which were not design criteria for IP-3. In response to these requests, we performed a scoping investigation which led us to conclude that realistic prediction of the effects of the postulated condition would require detailed inelastic system analysis. After reviewing the present analytical capabilities for dynamic inelastic analysis and the structural complexity of the Indian Point #3 primary system supporting structures, we have determined that analytical methods are not available today that would lead to a realistic and conclusive assessment of the reactor shutdown capability. Although a precise evaluation of the time, manpower and other resources needed to provide the requested information is not possible, it is estimated that it would take more than a year to develop the necessary models and calculational methods. Considering this, we have concluded that plant safety could be more confidently assured by augmenting the inservice inspection of the reactor vessel nozzle to pipe welds to assure the integrity of these welds.

We therefore propose an augmented inservice inspection program which would provide a degree of surveillance exceeding that required by the 1974 version of ASME B & PV Code Section XI, and which would give assurance through the life of the plant that

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