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REGULATORY DOCKET FILE COPY

November 26, 1975

Re Indian Point Unit No. 3  
Docket No. 50-286

Director of Nuclear Reactor Regulation  
Attn: Mr. R. C. DeYoung  
Assistant Director for  
Light Water Reactors  
Group 1

U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. DeYoung

On November 12, 1975 and November 17, 1975, we forwarded to you responses to the outstanding items listed in your letter of November 5, 1975. The purpose of the present submittal is to provide additional clarification on certain items as requested by members of your staff. This additional information is presented in the attachment.

Very truly yours

*RW Clement, Act V*  
for Carl L. Newman

enc.  
mk

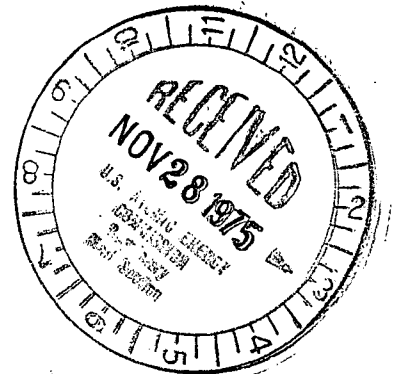
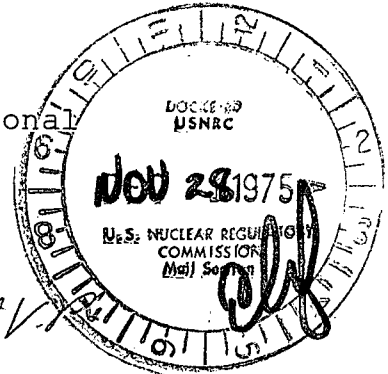
Sworn to before me this  
26 day of November, 1975.

WALTER S. HOSMER  
Notary Public State of New York  
No. 30-6973150  
Qualified in Nassau County  
Certificate filed in New York County  
Commission Expires March 30, 1976

*Walter S. Hosmer*

Notary Public

Copy to James P. O'Reilly, Director  
Office of Inspection and Enforcement  
Region 1  
U. S. Nuclear Regulatory Commission  
King of Prussia, Penn. 19406



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November 26, 1975

Re Indian Point Unit No. 3  
Docket No. 50-2861. Reactor Pressure Vessel Supports

Westinghouse is beginning a program of detailed analyses, specific to Indian Point Unit 3, to determine the loads in the reactor vessel support system and to evaluate the full restraint capability of the system for a postulated pipe rupture at the vessel inlet nozzle. These analyses will augment the information reported in our November 17, 1975 letter and will include the effect of the reactor vessel support stiffnesses, asymmetric cavity pressure loads, loads from the asymmetric forces in the vessel internals including hydroelastic coupling effects of the core barrel, and loads from attached coolant loops. The program will include refinement of methods used to determine the magnitudes and time histories of the applied loads. The effect of inelastic deformations that may occur in highly loaded members of the structure will be evaluated. It is our intention that the results of this work will be available by December 1, 1976.

2. Steam Generator Cladding

The following statement supplements the response given in our November 12, 1975 submittal to item 2b of enclosure 2 to your November 5, 1975 letter:

It is now proposed to accomplish the general 100% visual examination at each of the first three refueling shut-downs. In addition, representative surfaces will be examined with liquid penetrant and surface replication techniques, at each of the first three refueling shut-downs. A material such as RTV-11 silicone rubber will be used to obtain the replications for study, and measurements will be recorded.

### 3. Pressurization of Air Lock Seals

We are in the process of modifying both personnel airlocks to provide visual verification that the double seals on both the inner and outer doors of the airlocks are repressurized after the doors are reclosed.

In the original design, gauges were installed inside the containment to provide indication of seal pressure for the inner door and inside the airlock to provide indication of seal pressure for the outer door. These gauges will be retained. In addition, a gauge will be installed inside the airlock to provide indication of seal pressure for the inner door, and a gauge will be installed outside containment to provide indication of seal pressure for the outer door.

With the addition of the two gauges per airlock, capability is provided for visual verification of repressurization of both inner and outer door seals, regardless of whether a person is entering or exiting from containment.

Modifications are now underway and are scheduled for completion by November 28, 1975.