

ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATION CHANGES

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
AUGUST, 1978

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4.2.4

The inspection interval shall be ten years.

4.2.5

The following definitions shall apply to the inspection methods employed in Table 4.2-1. The paragraphs referenced are corresponding paragraphs of Section XI of the ASME Code for In-Service Inspection of Nuclear Reactor Coolant Systems dated January 1970, except as noted:

- a. UT - Ultrasonic examination per paragraph IS-213.2.*
- b. RT - Radiographic examination per paragraph IS-213.1.
- c. MT - Magnetic particle examination per paragraph IS-212.1.
- d. PT - Liquid penetrant examination per paragraph IS-212.2.
- e. V - Visual examination per paragraphs IS-211.1 or IS-211.2.
 - * All indications which produce a response greater than 100% of the reference level shall be investigated to the extent that the operator can determine the shape, identity, and location of all such reflectors and evaluate these indications per paragraph IS-311.
- f. Weld L-1 in the pressurizer shall utilize the inspection techniques and acceptance criteria of the Summer 1975 addenda to the 1974 Edition of Section XI of the ASME Code.

4.2.6

Examinations which reveal unacceptable structural defects in a category shall be extended to include an additional number (or areas) of system components or piping in the same category approximately equal to that initially examined. In the event further unacceptable structural defects are revealed, all remaining system components or piping in the category shall be examined to the extent specified in that examination category.

4.2.7

With the exception of those components or areas for which the examination may be deferred to the end of the inspection interval, at least 25 percent of the required examinations shall have been completed by

<u>Item No.</u>	<u>Examination Category</u>	<u>Components and Parts to be Examined</u>	<u>Method</u>	<u>Extent of Examination (Percent in 10 Year Interval)</u>	<u>Remarks</u>
		PRESSURIZER			outages shall be performed during each refueling period. Where access to the space below the reactor core during normal refueling outages precludes inspection of this space, at least one examination, at or near the end of each inspection interval, shall be conducted under conditions which enable inspection.
2.1	B	Longitudinal and circumferential welds.	V & UT	Longitudinal - 10% Circumferential - 5%	Accessibility of welds is limited by biological and missile shield. Longitudinal weld L1 shall be inspected during the second refueling outage using straight beam, 45° and 60° half-angle beam UT methods for its accessible length.
2.2	D	Nozzle-to-vessel welds	V & UT	See remarks	Instrument and sample nozzles are included in Item 2.4.
2.3	E-1	Heater connections		See remarks	These connections are considered in Item 2.4.

Amendment No.

ATTACHMENT II

SAFETY EVALUATION

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
AUGUST, 1978

Safety Evaluation
Primary Coolant System Pressurizer
Inspection of Weld L-1

Section I - Description of the Proposed Modification

The Power Authority wishes to augment the inspection requirements of the Primary Coolant System Pressurizer. Technical Specification §4.2.5 is being modified to apply the inspection techniques and acceptance criteria of the Summer 1975 addenda to the 1974 Edition of Section XI of the ASME Code to Weld L-1 in the PCS Pressurizer. Table 4.2-1 of the Technical Specifications is being modified to include straight beam, and 45° and 60° half-node angle beam UT inspection methods during the second refueling outage.

Section II - Purpose of the Modification

Presently the Indian Point Unit 3 Technical Specification requires Inservice Inspection in accordance with the ASME Boiler and Pressure Vessel Code Section XI, 1970 edition. The Inservice Inspection for the first refueling outage has been conducted in accordance with the 1970 edition of the code and a recordable indication was found at the Pressurizer Weld L-1. However, since the 1970 code has no acceptance parameters for such an indication, it is the purpose of this modification to allow examination of longitudinal weld L-1 in accordance to ASME Section XI, 1974 edition (including addenda to Summer, '75.) This edition of the code contains specific evaluation parameters in Table IWB 3511.1 and IWB 3511.2.

Section III - Impact of Change of FSAR

A review of the FSAR and SER accident analyses indicates that there are no changes to the parameters and assumptions used and that the conclusions reached are unchanged as a result of this modification to the Technical Specifications.

During the Inservice Inspection in accordance with the 1970 edition of ASME Section XI Longitudinal Weld L-1 was examined by 45° angle beam in accordance with code. No recordable indications were noted. The weld was also examined utilizing straight beam examination. By straight beam examination, an indication approximately 4.1 inches by .5 inches wide was noted. Further tests indicated that it was laminar in nature, in length parallel to the weld, and approximately 2 inches below the outside diameter surface of the weld.

The weld history was then reviewed. Radiographs of the as-fabricated weld were carefully examined and no defects were seen. Baseline ultrasonic testing data for weld L-1 was reviewed and the baseline examination contained no reportable indications. The baseline examination was done in accordance with the 1970 code and only angle beam techniques were used.

It is strongly believed that the indication was present as-fabricated, however, because of its orientation and almost insignificant thickness it would not be expected to appear on the radiographs.

Since the 1970 edition of Section XI has no acceptance parameters for such an indication it was decided to re-examine the weld in accordance with the ASME Code Section XI, 1974 edition, including the addenda to the summer of '75.

This examination was conducted utilizing straight beam, 45° angle beam, and 60° angle beam methods (half-node). The accessible length of the weld was scanned by all three (3) methods. There were no recordable indications by 45° and 60° angle beam examinations. However, straight beam examination again revealed the indication mentioned above.

The Summer '75 addenda of ASME Section XI has acceptance parameters for both Planar and Laminar conditions in Tables IWB 3511.1 and IWB 3511.2. Calculations show that the indication found in L-1 by straight beam examination would be acceptable to either of these tables regardless of whether the indication is characterized as Planar or Laminar.

Section IV-Implementation of Modification

The modification as proposed will not impact the ALARA and Fire Protection Programs.

Section V - Conclusions

(a) The probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report has not been increased; (b) a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis reports has not been created; and (c) the margin of safety as defined in the basis for any technical specifications is not reduced.

Section VI - References

- (a) IP3 FSAR
- (b) IP3 Safety Evaluation Reports