

ATTACHMENT A

APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

Consolidated Edison Company of New York, Inc.

Power Authority of the State of New York

Indian Point 3 Nuclear Power Plant

Docket No. 50-286

Facility Operating License No. DRP-64

August 12, 1977

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1.13 SURVEILLANCE INTERVAL

When Refueling Outage is used to designate a surveillance interval, the surveillance interval shall not exceed 18 months, except for the first fuel cycle. The first refueling outage surveillance testing will be performed during the first refueling outage.

Surveillance intervals, with the exception of refueling, shift and daily periods, are defined as the specified period plus or minus 25% of the specified period.

E. Containment Isolation Valves

1. Tests and Frequency

- a. Isolation valves in Table 4.4-1 shall be tested for operability at intervals no greater than 2 years.
- b. Isolation valves in Table 4.4-1 which are pressurized by the Weld Channel and Penetration Pressurization System shall be leakage tested as part of the Weld Channel and Penetration Pressurization System Test at intervals no greater than 2 years.
- c. Isolation valves in Table 4.4-1 which are pressurized by the Isolation Valve Seal Water System shall be tested at intervals no greater than 2 years as part of an overall Isolation Valve Seal Water System Test.
- d. Isolation valves in Table 4.4-1 which are not pressurized will be tested at intervals no greater than 2 years.
- e. Isolation valves in Table 4.4-1 shall be tested with the medium and at the pressure specified therein.

2. Acceptance Criteria

- a. The combined leakage rate for the following shall be less than $0.6 L_a$: isolation valves listed in Table 4.4-1 subject to gas pressurization testing, air lock testing as specified in D.1, portions of the sensitive leakage rate test described in C.1 which pertain to containment penetrations and double-gasketed seals.
- b. The leakage rate into containment for the isolation valves sealed with the service water system is 0.36 gpm per fan cooler.

ATTACHMENT B

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SAFETY EVALUATION

The proposed change to Indian Point 3 Technical Specification Section 1.13, contained in Attachment A to this Application, would clarify the intent of that specification with regard to its applicability to refueling outage surveillance tests performed at the end of the first fuel cycle. Surveillance tests designated to be performed at a refueling outage are so designated because these tests can only be performed when the plant is shutdown. The length of a fuel cycle is normally less than the 18 month maximum interval specified, except for the first fuel cycle which generally runs somewhat longer. The first refueling outage surveillance test will be performed at the first refueling outage.

The proposed change to Indian Point 3 Technical Specification Section 4.4.E.1, contained in Attachment A to this Application, would render the technical specification requirement for testing of containment isolation valves consistent with the requirements of Appendix J to Part 50 of the Commission's regulations, which allows a two year interval between surveillance tests of containment isolation valves.

The proposed changes do not in any way alter the safety analyses performed in the FSAR for Indian Point Unit No. 3. The proposed changes have been reviewed by the Station Nuclear Safety Committee and the Con Edison Nuclear Facilities Safety Committee. Both committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types of or increase in the amounts of effluents or any change in the authorized power level of the facility.