

William J. Cahill, Jr.
President

Consolidated Edison Company of New York, Inc.
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June 10, 1977
Indian Point Station
Docket Nos. 50-247
50-286

Mr. Eldon J. Brunner, Chief
Reactor Operations and Nuclear Support Branch
U. S. Nuclear Regulatory Commission, Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Brunner

This refers to Inspection 50-247/77-10 and 50-286/77-11 conducted by your Mr. T. Rebelowski on April 18-22, 1977 of activities authorized by NRC License Nos. DPR-26 and DPR-64 at our Indian Point Station. Your May 20, 1977 letter stated that it appeared that one of our activities was not conducted in full compliance with NRC requirements. This particular activity was set forth in the Notice of Violation enclosed as Appendix A to your letter. Our response to this item of apparent non-compliance is as follows:

On January 3, 1976, excessive stem packing leakage from the motor operated valve (MOV-731) in the RHR suction line from No. 22 reactor coolant loop necessitated a shutdown of the Unit. In accordance with the requirements of Technical Specification 3.1.F.3, a safety evaluation was made and documented for this occurrence. When a similar event occurred on September 27, 1976, a safety evaluation was not documented as it was felt that the January 3, 1976 evaluation was applicable. However, a safety evaluation for this event has now been documented.

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In order to avoid further items of non-compliance of this type, we have instructed our licensed operators that documentation of the safety evaluations required by Technical Specification 3.1.F.3 (now 3.1.F.6) are necessary for each event, no matter how similar in nature they may appear to be.

Very truly yours

William J. Bell, Jr.