January 13, 2010

Mr. Normal A. Kent, Manager
Licensing, Compliance, and Package Technology
Westinghouse Electric Company LLC
P.O. Drawer R
Columbia, South Carolina 29250

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR REVIEW OF THE MODEL NOS. TRAVELLER STD AND TRAVELLER XL TRANSPORTATION PACKAGE

Dear Mr. Kent:

By application dated October 23, 2009, Westinghouse Electric Company submitted a request to the U.S. Nuclear Regulatory Commission for renewal of Certificate of Compliance No. 9297. The application requests that the package review include consideration of NUREG-1886, "Joint Canada – United States Guide for Approval of Type B(U) and Fissile Material Transportation Packages."

In connection with our review, we need the information identified in the enclosure to this letter. To assist us in scheduling staff review of your response, we request that you provide this information by February 12, 2010. If you are unable to provide a response by that date, our review may be delayed. To assist us in re-scheduling your review, you should include a new proposed submittal date and the reasons for the delay.

Please reference Docket No. 71-9297 and TAC No. L24387 in future correspondence related to this request. The staff is available to meet to discuss your proposed responses. If you have any questions regarding this matter, I may be contacted at (301) 492-3292.

Sincerely,

/RA/

Michele M. Sampson, Sr. Project Manager
Licensing Branch
Division of Spent Fuel Storage and Transportation
Office of Nuclear Material Safety
and Safeguards

Docket No. 71-9297
TAC No. L24387

Enclosure: Request for Additional Information

cc w/encl: E. Redmond, NEI

January 13, 2010

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Request for Additional Information
Westinghouse Electric Company LLC
Docket No. 71-9297
Certificate of Compliance No. 9297
Model Nos. Traveller STD and Traveller XL

By application dated October 23, 2009, Westinghouse Electric Company submitted a request to the U.S. Nuclear Regulatory Commission for renewal of Certificate of Compliance No. 9297. The application requests that the package review include consideration of NUREG-1886, "Joint Canada – United States Guide for Approval of Type B(U) and Fissile Material Transportation Packages." This request for additional information (RAI) identifies information needed by the U.S. Nuclear Regulatory Commission (NRC) staff in connection with its review of the amendment. The requested information is listed by chapter number and title in the applicant's Safety Analysis Report.

Each individual RAI describes information needed by the staff for it to complete its review of the application and/or the SAR and to determine whether the applicant has demonstrated compliance with the regulatory requirements of 10 Code of Federal Regulations Part 71 (10 CFR Part 71) and TS-R-1, 1996 Edition (As Amended 2003). NUREG-1609, "Standard Review Plan for Transportation Packages for Radioactive Material," and NUREG-1886, were used by the staff in its review of the application.

1-0 GENERAL INFORMATION

1-1 Provide a generic sketch representing the package as prepared for transport.

The appendix to Chapter 1 of the Traveller SAR includes the engineering drawings, but does not have a generic sketch which represents the package as prepared for transport. This sketch should be a reproducible illustration, not larger than 21 cm by 30 cm, showing the make-up of the package.

This information is required to assess compliance with TS-R-1, para. 807(h).

2-0 STRUCTURAL EVALUATION

2-1 Revise Sections 2.6.3 and 2.6.4, of the SAR to provide consistent information related to the evaluation of the package under both reduced and increased external pressure.

Currently, the SAR in Sections 2.6.3 and 2.6.4, states that Appendix 2.12.2 presents calculations demonstrating the package performance under reduced and increased external pressure. However, Appendix 2.12.2 does not contain calculations showing the performance of the package if it were sealed. Sections 2.6.3, 2.6.4, and Appendix 2.12.2.2.4.1, contain a reasoned argument that the design of the package precludes development of a pressure differential. This reasoned argument forms the basis for staff's findings for 10 CFR 71.71(c)(3) and (4), in the Safety Evaluation Report issued with Revision 0 of the Certificate [ML050750168].

This information is required to assess compliance with 10 CFR 71.71(c)(3) and (4).

2-2 Provide justification that the sequence for the drop tests represents the order which results in the specimen suffering such damage as will lead to maximum damage in the thermal test.

Chapter 2 of the SAR does not provide a discussion of the selection of drop test order related to the assessment of maximum damage. The order for the puncture and 9-meter drop test should be evaluated to confirm that the order in which the tests were performed provides the maximum damage.

This information is required to assess compliance with TS-R-1, para. 727.