January 22, 2010

LICENSEE: Arizona Public Service Company

FACILITY: Palo Verde Nuclear Generating Station, Units 1, 2, and 3

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JANUARY 14, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ARIZONA PUBLIC SERVICE COMPANY, CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION PERTAINING TO THE PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (the staff) and representatives of Arizona Public Service Company (the applicant) held a telephone conference call on January 14, 2010, to discuss and clarify the staff's draft request for additional information (RAI) concerning the Palo Verde Nuclear Generating Station, Units 1, 2, and 3, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's draft RAI.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the draft questions discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/

Lisa M. Regner, Sr. Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-528, 50-529, and 50-530

Enclosures:
As stated

cc w/encls: See next page

January 22, 2010

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SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JANUARY 14, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ARIZONA PUBLIC SERVICE COMPANY, CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION PERTAINING TO THE PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3, LICENSE RENEWAL APPLICATION

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The applicant had an opportunity to comment on this summary.

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Docket Nos. 50-528, 50-529, and 50-530

Enclosures:
As stated

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DATE	01/19/10	01/20/10	01/22/10	01/22/10

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Memo to Arizona Public Service Company from Lisa M. Regner dated January, 22, 2010

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON JANUARY 14, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ARIZONA PUBLIC SERVICE COMPANY, CONCERNING DRAFT REQUEST FOR ADDITIONAL INFORMATION PERTAINING TO THE PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3, LICENSE RENEWAL APPLICATION

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TELEPHONE CONFERENCE CALL
PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS
JANUARY 14, 2010

PARTICIPANTS	**AFFILIATIONS**
Lisa Regner	U.S. Nuclear Regulatory Commission (NRC)
Albert Wong	NRC
Todd Mintz	Center for Nuclear Waste Regulatory Analyses (CNWRA)
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Glenn Michael	APS
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Donald Stevens	STARS
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ENCLOSURE 1

TELEPHONE CONFERENCE CALL CONCERNING
DRAFT REQUEST FOR ADDITIONAL INFORMATION
PALO VERDE NUCLEAR GENERATING STATION, UNITS 1, 2, AND 3
LICENSE RENEWAL APPLICATION

JANUARY 14, 2010

The U.S. Nuclear Regulatory Commission (the staff) and representatives of Arizona Public Service Company (the applicant) held a telephone conference call on January 14, 2010, to discuss and clarify the following draft request for additional information (RAI) concerning the Palo Verde Nuclear Generating Station, Units 1, 2, and 3, license renewal application (LRA).

DRAFT RAI 3.1.2-1

Background
The Generic Aging Lessons Learned (GALL) Report, Section X.M1 recommends the Metal Fatigue of Reactor Coolant Pressure Boundary Aging Management Program (AMP) for managing the aging of selected reactor coolant system components.

Issue
In the LRA, Section 3.1.2.2.1, the applicant states that the pressurizer relief tank is not an ASME Section III Class 1 component, nor is it designated to other fatigue or cycle design rules, and therefore has no fatigue time-limited aging analysis (TLAA). However, the GALL Report, Section IV, Subsection C2, item IV.C2-23 states that a time-limited aging analysis is needed to evaluate metal fatigue of the pressurizer relief tank for the period of extended operation.

Request
Provide additional information how metal fatigue of the pressurizer relief tank will be evaluated and managed for the period of extended operation.

DRAFT RAI 3.2.2-1

Background

The GALL Report, Table V.C, item V.C-4, indicates that stainless steel containment isolation piping and component internal surfaces exposed to treated water are subject to loss of material/pitting and crevice corrosion as an aging effect/mechanism. The GALL Report states that the aging effect/mechanism can be managed by the Water Chemistry and One-time Inspection Programs. The GALL Report further states that a further evaluation should be conducted on these components.

Issue

Section 3.2.2.2.8 of the LRA indicates that the further evaluation associated with this component is not applicable because the containment isolation components were “evaluated in the systems in which the components were found to have the function of containment integrity.” Staff is unclear why the applicant indicates that this further evaluation for steel containment isolation piping is not applicable.

Request

Provide additional information what is meant by the containment isolation components were evaluated in the system in which the components were found to have the function of containment integrity. In addition, if a further evaluation is conducted for these components in a different system, provide the location of the further evaluation. If further evaluations were not conducted for these components, provide additional justification.

DRAFT RAI 3.3.2-1

Background

The GALL Report, Table VIII, item VIII.A-5 is for copper piping exposed to treated water in the steam turbine system. The GALL Report suggests the use of the Water Chemistry Program augmented with the One-Time Inspection Program to manage loss of material and selective leaching in this system. However, the GALL Report in Table VII, item VII.C2-7 is for copper piping (Zinc > 15 percent) exposed to treated water in the Closed-Cycle Cooling Water System. The GALL Report suggests that loss of material and selective leaching for this component and system be managed by the Selective Leaching of Materials Program. One of the main differences between these two programs as described in the GALL Report is that hardness measurements are incorporated into the Selective Leaching of Materials Program, not in the Water Chemistry Program.

Issue

Table 3.3.2-4 of the LRA indicates that a copper alloy (Zinc > 15 percent) piping exposed to demineralized water is consistent with GALL Report item VIII.A-5. However, LRA Table 3.3.2-4 is a closed-cycle cooling water system. As such it seems that the GALL Report item VII.C2-7 may be more appropriate for this material. It is not clear to the staff how the Water Chemistry Program augmented with the One-Time Inspection Program is sufficient to manage the loss of material for the LRA closed-cycle cooling water component since these programs do not require hardness testing.

Request

Provide additional explanation on how the Water Chemistry Program (B2.1.2) and the One-Time Inspection (B2.1.16) will address the aging effect for loss of material for the copper alloy piping exposed to demineralized water in a Closed-Cycle Cooling Water System.

Discussion: The applicant indicated that all questions are clear. These draft questions will be sent in a formal RAI.

Palo Verde Nuclear Generating Station,
Units 1, 2, and 3

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