

Rel Cohen
50-247

LAW OFFICES OF
LEBOEUF, LAMB, LEIBY & MACRAE
1757 N STREET, N.W.
WASHINGTON, D.C. 20036

March 4, 1974

ONE CHASE MANHATTAN PLAZA
NEW YORK, N.Y. 10005

WASHINGTON TELEPHONE
202-672-6668

CABLE ADDRESS
LALALU, WASHINGTON D.C.

ARVIN E. UPTON
EUGENE B. THOMAS, JR.
LEONARD M. TROSTEN
HARRY H. VOIGT
LEX K. LARSON
WASHINGTON PARTNERS

Anthony Z. Roisman, Esq.
Berlin, Roisman & Kessler
Fourth Floor
1712 N Street, N.W.
Washington, D.C. 20036

Re: Indian Point 2

Dear Mr. Roisman:

Enclosed is a copy of a letter from Mr. Cobean
to Mr. O'Reilly dated February 26, 1974.

Very truly yours,

LEBOEUF, LAMB, LEIBY & MACRAE
Attorneys for Consolidated Edison
Company of New York, Inc.

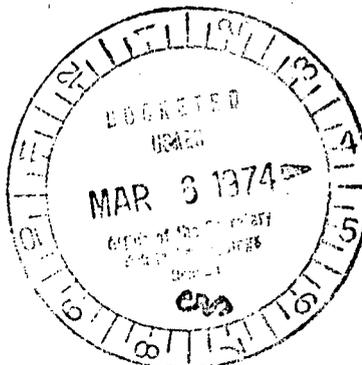
BY

Edward L. Cohen
Edward L. Cohen

BY HAND

Enclosure

cc w/enc: William C. Parler, Esq.
Dr. John A. Buck
Dr. Lawrence R. Quarles
Samuel W. Jensch, Esq.
Mr. R.B. Briggs
Dr. John C. Geyer
Myron Karman, Esq.
Angus Macbeth, Esq.
J. Bruce MacDonald, Esq.
Hon. Louis J. Lefkowitz
Secretary, U.S. Atomic Energy Commission
Atomic Safety and Licensing Board Panel



8110240196 740304
PDR ADDCK 05000247
S PDR

Eastern memorandum

15x copies
50-247

February 26, 1974

Re: Indian Point Unit No. 2
AEC Docket No. 50-247
Operating License DPR-26
A.O. 4-2-9

Mr. James P. O'Reilly, Director
Regulatory Operations, Region I
U. S. Atomic Energy Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

LoBuef, Lamb, Leiby & MacRae

RECEIVED
MAR 1 1974

Per *[Signature]*

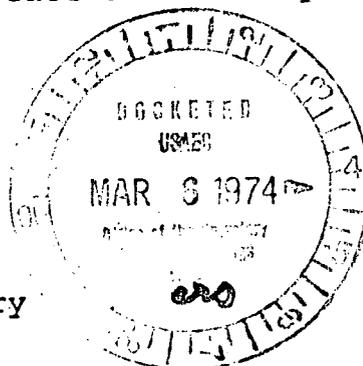
Dear Mr. O'Reilly:

In accordance with the requirements of Section 6.12.2(a) of the Technical Specifications of Facility Operating License No. DPR-26, the following report is submitted:

In the course of performing periodic surveillance test PT-M11, "Steam Line Pressure Analog Channel Functional Test" on February 22, 1974, an inadvertent safety injection signal was generated which, by design, caused the accumulator discharge stop valves to open. At the time of the occurrence, the reactor was in the cold shutdown condition with the Residual Heat Removal System in service and a reactor coolant pressure and temperature of 150 psig and 1150F respectively. Since the reactor coolant system was being operated in the solid mode, opening of the accumulator valves pressurized the system to about 560 psig which was slightly in excess of the Technical Specification limit of 500 psig for indicated temperatures at or below 2200F. The pressure was promptly reduced below the 500 psig limit by operator action.

There was no damage incurred to any system or component as a result of the pressure transient of this magnitude nor was there any reason to expect any.

Mr. Anthony Fasano of your office was informed of this occurrence by Mr. John M. Makepeace on February 22, 1974.



Very truly yours,

Warren R. Cobean, Jr.

Warren R. Cobean, Jr. Manager
Nuclear Power Generation Department

cc: John F. O'Leary