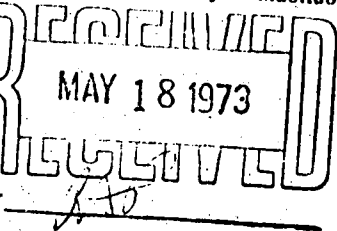


e Boeuf, Lamb, Leiby & MacRae



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May 16, 1973

Re: Indian Point Unit No. 2
Docket No. 50-247
Facility Operating
License DPR-26

Mr. James P. O'Reilly, Director
Regulatory Operations, Region 1
U. S. Atomic Energy Commission
970 Broad Street
Newark, New Jersey 07102

Dear Mr. O'Reilly:

Page 1 of 4

Send the above message, subject to the terms on back hereof, which are hereby agreed to

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In accordance with the requirements of Technical Specification 6.6.1.B, we wish to inform you of an abnormal occurrence which was identified on May 15, 1973 at 1300 hours.

During the course of a routine plant status inspection, a slight leak was observed at a weld on a 3/4 inch pipe to socket connection between Valve S-47 and Line No. 355 of the Residual Heat Removal System. The connection is located on the discharge side of the Residual Heat Removal Exchangers, upstream of Check Valve 838A. The reactor at the time of the occurrence was in the cold shutdown condition at atmospheric pressure.

Page 2 of 4

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PDR JOCK 05000247
Q PDR

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In accordance with the requirements of Technical Specification 6.2, notification of the Station Manager and the Chairman of the Nuclear Facilities Safety Committee has taken place. Mr. A Fasano of your office was also notified on May 16, 1973 of the occurrence by Mr. J. Makepeace, Chief Engineer, Unit No. 2.

The safety implications of the occurrence are minimal in that the reactor was in the cold shutdown condition and depressurized. In addition, the leak was well within the makeup capability of the Chemical and Volume Control System. Although initial criticality has not yet taken place, the leakage, even if it had been radioactive, would have been detected and contained

Page 3 of 4

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within the vapor container resulting in no danger to the health and safety of the public.

We are currently investigating the cause of the leak and will submit the required follow-up report within ten (10) days.

Very truly yours

William E. Caldwell, Jr.
Vice President

Page 4 of 4

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