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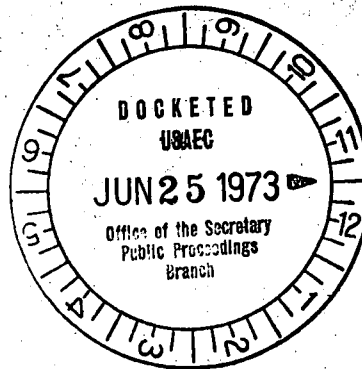
June 21, 1973

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Board
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Washington, D.C. 20036



Re: Consolidated Edison Company
of New York, Inc.
Indian Point Unit No. 2
AEC Docket No. 50-247

Dear Mr. Chairman:

In accordance with the stipulation among the parties (Tr. 9852-55) we hereby submit on behalf of Consolidated Edison Company of New York, Inc. a document entitled "50-Day Report of the Indian Point Unit No. 2 Testing Program" and dated June 19, 1973. This report sets forth the progress of the testing program at Indian Point 2 through June 9, 1973.

Very truly yours,

LEBOEUF, LAMB, LEIBY & MACRAE
Attorneys for Applicant

By Leonard M. Trosten
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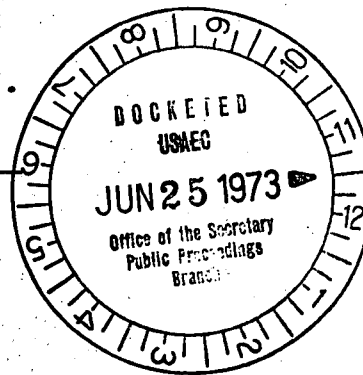
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Enclosures

cc w/encs: see attached

cc w/enclosures: Mr. R. B. Briggs
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Honorable Louis J. Lefkowitz
Secretary, U.S. Atomic
Energy Commission
J. Bruce MacDonald, Esq.
Atomic Safety and Licensing Board Panel

Consolidated Edison Company of New York, Inc.
Indian Point Unit No. 2
Docket No. 50-247



50-DAY REPORT OF THE INDIAN POINT
UNIT NO. 2 TESTING PROGRAM

Amendments Nos. 1 and 2 to Facility Operating License No. DPR-26 issued to Consolidated Edison Company of New York, Inc. on April 20, 1973 and April 27, 1973 authorized the operation of Indian Point Unit No. 2 for testing purposes at power levels not to exceed 20% and 50% of rated power, respectively. After the issuance of Amendment No. 1, Consolidated Edison completed its subcritical testing program and prepared for initial criticality and subsequent testing of the facility. Preparation for criticality consisted mainly of performing the Reactor Coolant System heatup sequence and testing. The heatup sequence required successful completion of checkoff lists and instrument tests. The major pre-critical tests performed include:

1. Cold Control Rod Stepping Tests and Hot Rod Drop Tests

These tests confirmed that control rod movement is satisfactory.

2. Main Steam Safety Valve Tests

These tests confirmed that modifications made to the main steam safety valve installation were successful.

3. Four-out-of-Four Reactor Coolant Pump Coastdown Test

This test confirmed the curve presented in Figure 6.3 of the report "Fuel Densification - Indian Point Unit No. 2" (Westinghouse non-proprietary) dated January 1973 which was used as input to the analysis of the loss of flow transient.

Consolidated Edison had scheduled initial criticality to take place during April 1973. Criticality during April was delayed, however, when the results of the Reactor Coolant System hydrostatic integrity test revealed several leaks in valve packings and flanges. Although acceptable indentified leakage is allowed in accordance with Technical Specification limits, Consolidated Edison deemed that it would be prudent to eliminate such leakage prior to criticality. This effort consisted of replacing valve packing and tightening of flanges. Reactor Coolant System heatup and hydrostatic testing were repeated several times to identify leaks and to verify steam integrity following corrective actions. One two-inch valve in the CVCS changing line had minor leakage through two stud holes and was replaced, and one vent assembly in the RHR system required additional weld repair.

Following the foregoing corrective actions, the reactor was taken critical for the first time on May 22, 1973 at 2:39 p.m. Since initial criticality, the reactor was taken critical on

several occasions at power levels up to approximately 4 percent of full power in order to conduct physics tests and equipment checkout as part of the "zero power" phase of the testing program. The zero power phase of the testing program was successfully completed during the week of June 4, 1973. The significant tests conducted during the zero power phase of the testing program were:

1. Dynamic Control Rod Worth Measurements -
(differential and integral worth for all control and shutdown banks, including part-length control rods). The results of these tests confirmed that the rod worths are within the acceptable range to support the analyses presented in Applicant's FSAR and Fuel Densification Report.
2. Dynamic Temperature Coefficient Measurements -
(includes moderator temperature coefficient and Doppler coefficient).
These tests confirmed the as-analyzed core design at essentially zero-power.
3. Core-wide Neutron Flux Mapping - This test confirmed proper core loading, power distribution at essentially zero power and operation of movable in-core instrumentation.

4. Dynamic Boron Worth Measurements - These tests consisted of measuring required boron concentrations in the primary coolant for maintaining criticality at various rod configurations. The results agreed favorably with values predicted for the as-analyzed core design.
5. Minimum Shutdown Verification - This test determined the minimum shutdown boron concentration required to maintain the shutdown margin as specified in the Technical Specifications.

On June 8, 1973, steam was admitted to the main steam system for testing of various secondary system components at reactor power levels up to 20% of full power, and the turbine-generator was brought up to rated speed in preparation for synchronization of the main generator to the system. Synchronization did not take place due to the need for adjustments of turbine controls. The reactor was shut down in order to complete such adjustments. At this time the reactor was brought to the cold shutdown condition in order to perform maintenance on the valve seats of the pressurizer safety and relief valves so as to eliminate leakage observed across these valves to the pressurizer relief tank.