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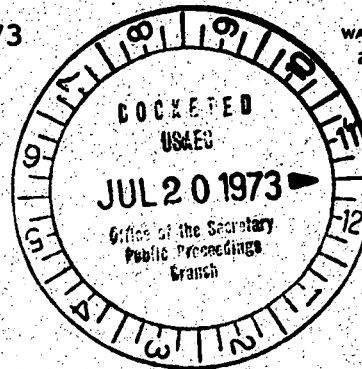
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July 18, 1973



Anthony Z. Roisman, Esq.  
Berlin, Roisman & Kessler  
Fourth Floor  
1712 N Street, N.W.  
Washington, D. C. 20036

Re: Consolidated Edison Company  
of New York, Inc.  
Indian Point Unit No. 2  
AEC Docket No. 50-247

Dear Mr. Roisman:

Enclosed are copies of a telegram from Consolidated Edison to Mr. O'Reilly dated July 3, 1973 and a letter from Mr. Zarakas to Mr. O'Leary dated July 12, 1973. A copy of the text of the July 3 telegram which was received in our office yesterday is being transmitted since a copy of the original telegram has not yet been received.

Very truly yours,

LEBOEUF, LAMB, LEIBY & MACRAE  
Attorneys for Applicant

By Edward L. Cohen  
Edward L. Cohen

BY HAND

Enclosures

cc w/encs: See next page

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- 2 -

cc w/encs:

Samuel W. Jensch, Esq.

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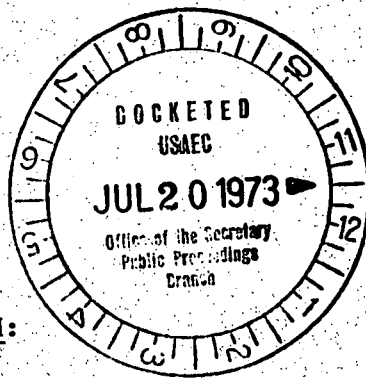
Angus Macbeth, Esq.

J. Bruce MacDonald, Esq.

Honorable Louis J. Lefkowitz

Secretary, U. S. Atomic Energy Commission (2)

Atomic Safety and Licensing Board Panel



July 3, 1973

TEXT OF TELEGRAM:

Re: Indian Point Unit No. 2  
Facility Operating  
License DPR-26

Mr. James P. O'Reilly, Director  
Regulatory Operations  
Region 1  
U.S. Atomic Energy Commission  
970 Broad Street  
Newark, New Jersey 07102

Dear Mr. O'Reilly:

In accordance with the requirements of Technical Specification 6.6.1.B, the following abnormal occurrence report (A.O. 3-2-9) is provided: During the performance of periodic test and calibration procedure PTM-12, on July 2, 1973, it was discovered that the first stage turbine pressure setting was slightly in excess of the limit defined by Facility Technical Specification 2.3.2.A.2. Instead of the trip becoming unblocked at the prescribed 10% of full power maximum, one of the two channels was found to be set at 10.3%. Additionally, the periodic test revealed that one of eight safety injection steam flow bi-stables was set 0.3% higher than that prescribed in Table 3.1 of the Technical Specifications. The proper settings for these two devices were established during the course of the calibration check.

An investigation as to the cause of the setting deviations is presently being conducted. The results of the investigation and any measures taken to prevent recurrence will be reported in a letter provided within ten (10) days as required by the facility license.

In accordance with the requirements of Technical Specification 6.2, notification to the Chairman of the Nuclear Facilities Safety Committee has taken place.

- 2 -

Mr. C. Oberg of the U.S. Atomic Energy Commission, Region 1, Regulatory Operations, was also notified by telephone of the occurrence by Mr. John M. Makepeace, Chief Engineer of Unit No. 2, on July 2, 1973.