



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

February 26, 2010

Mr. David A. Heacock  
President and Chief Nuclear Officer  
Virginia Electric and Power Company  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT NOS. 1 AND 2, ISSUANCE OF AMENDMENT REGARDING REQUEST TO CORRECT TECHNICAL SPECIFICATION TABLE 3.7.1 OPERATOR ACTION 3.b (TAC NOS. ME1068 AND ME1069)

Dear Mr. Heacock:

The U.S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 268 to Renewed Facility Operating License No. DPR-32 and Amendment No. 267 to Renewed Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2 (Surry Units 1 and 2), respectively. The amendments change the Technical Specifications (TSs) in response to your application dated April 13, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML091040147).

These amendments revise the TSs of Surry Units 1 and 2. The proposed change revises TS Table 3.7.1, Operator Action 3.b, and provides direction for the actions to be taken if the operating condition of fewer than the required minimum channels for the neutron flux intermediate range occurs between 7 percent and 11 percent of rated power.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in cursive script that reads "Karen Cotton".

Karen Cotton, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosures:

1. Amendment No. 268 to DPR-32
2. Amendment No. 267 to DPR-37
3. Safety Evaluation

cc w/encls: Distribution via Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 268  
Renewed License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated April 13, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 268 , are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Gloria J. Kulesa, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to License No. DPR-32  
and the Technical Specifications

Date of Issuance: February 26, 2010



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 267  
Renewed License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated April 13, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Renewed Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 267 , are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Gloria J. Kulesa, Chief  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes License No. DPR-37  
and the Technical Specifications

Date of Issuance: February 26, 2010

ATTACHMENT

TO LICENSE AMENDMENT NO. 268

RENEWED FACILITY OPERATING LICENSE NO. DPR-32

DOCKET NO. 50-280

AND

TO LICENSE AMENDMENT NO. 267

RENEWED FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Replace the following pages of the Licenses and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

License

License No. DPR-32, page 3  
License No. DPR-37, page 3

TSs

Units 1 and 2  
TS 3.7.14

Insert Pages

License

License No. DPR-32, page 3  
License No. DPR-37, page 3

TSs

Units 1 and 2  
TS 3.7.14

3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2548 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 268 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. Deleted by Amendment 65

F. Deleted by Amendment 71

G. Deleted by Amendment 227

H. Deleted by Amendment 227

I. Fire Protection

The licensee shall implement and maintain in effect the provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report and as approved in the SER dated September 19, 1979, (and Supplements dated May 29, 1980, October 9, 1980, December 18, 1980, February 13, 1981, December 4, 1981, April 27, 1982, November 18, 1982, January 17, 1984, February 25, 1988, and

- E. Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:
- A. Maximum Power Level  
The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2546 megawatts (thermal).
  - B. Technical Specifications  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 267, are hereby incorporated in this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.
  - C. Reports  
The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.
  - D. Records  
The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.
  - E. Deleted by Amendment 54
  - F. Deleted by Amendment 59 and Amendment 65
  - G. Deleted by Amendment 227
  - H. Deleted by Amendment 227

TABLE 3.7-1 (Continued)

4. The QUADRANT POWER TILT shall be determined to be within the limit when above 75 percent of RATED POWER with one Power Range Channel inoperable by using the moveable incore detectors to confirm that the normalized symmetric power distribution, obtained from 2 sets of 4 symmetric thimble locations or a full-core flux map, is consistent with the indicated QUADRANT POWER TILT at least once per 12 hours.

With the number of OPERABLE channels one less than required by the Minimum OPERABLE Channels requirement, be in at least HOT SHUTDOWN within 6 hours

ACTION 3.

With the number of OPERABLE channels one less than required by the Minimum OPERABLE Channels requirement and with the THERMAL POWER level:

- a. Below the P-6 (Block of Source Range Reactor Trip) setpoint, restore the inoperable channel to OPERABLE status prior to increasing THERMAL POWER above the P-6 Setpoint.
- b. Above the P-6 (Block of Source Range Reactor Trip) setpoint, but below 11% of RATED POWER, within 24 hours, decrease power below P-6 or increase THERMAL POWER above 11% of RATED POWER.
- c. Above 11% of RATED POWER, POWER OPERATION may continue.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 268 TO

RENEWED FACILITY OPERATING LICENSE NO. DPR-32

AND

AMENDMENT NO. 267 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated April 13, 2009, Virginia Electric and Power Company (Dominion, the licensee), submitted a license amendment request (LAR) to revise the technical specifications (TSs) of Surry Power Station, Unit Nos. 1 and 2 (Surry Units 1 and 2). The proposed change revises TS Table 3.7.1, Operator Action 3.b, and provides direction for the actions to be taken if the operating condition of less than the required minimum channels for the neutron flux intermediate range occurs between 7 percent and 11 percent of rated power. The proposed change will revise TS Table 3.7.1, Operator Action 3.b, to read, "Above the P-6 (Block of Source Range Reactor Trip) setpoint, but below 11% of RATED POWER, within 24 hours decrease power below P-6 or increase THERMAL POWER above 11% of RATED POWER." The text currently reads, "Above the P-6 (Block of Source Range Reactor Trip) setpoint, but below 7% of RATED POWER, decrease power below P-6 or, increase THERMAL POWER above 11% of RATED POWER within 24 hours."

2.0 REGULATORY EVALUATION

The following guidance document pertains to the proposed TS change:

NRC Administrative Letter (AL) 98-10, "Dispositioning of Technical Specifications That Are Insufficient To Assure Plant Safety," dated December 29, 1998, states, "Imposing administrative controls in response to an improper or inadequate TS is considered an acceptable short-term corrective action." AL 98-10 also requires licensees to submit such a TS change with appropriate justification in a timely manner.

3.0 TECHNICAL EVALUATION

The licensee's request was based on a concern raised during a simulator training session that TS Table 3.7.1 Operator Action 3.b, as written, does not specify actions to be taken if the condition of

having less than the required minimum number of operable channels for the nuclear flux intermediate range occurs between 7 percent and 11 percent of rated power. Therefore, the actions directed by Operator Action 3.b are inadequate. As such, the licensee, consistent with the guidance of AL 98-10, imposed administrative controls by revising the Surry Units 1 and 2 Abnormal Procedures for Nuclear Instrumentation Malfunction for nuclear flux intermediate range channel inoperability for rated power levels between 7 percent and 11 percent.

The proposed TS change does not involve the installation of any new equipment or affect the design function of any structures, systems, or components (SSCs). In addition, the proposed change does not affect the condition or performance of SSCs relied upon for accident mitigation or impact any safety analysis assumptions. The licensee has taken proper actions in accordance with AL 98-10 to correct inadequate TS requirements; therefore, the NRC staff finds the proposed change acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (74 FR 34049). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

Based on its review of the licensee's submittal, the staff has concluded that the licensee has taken actions in accordance with AL 98-10 and has revised the TSs to correct the inadequate TS requirements. Therefore, the NRC Staff finds the proposed change acceptable.

Principal Contributor: Hukam Garg, NRR/DE

Date: February 26, 2010

February 26, 2010

Mr. David A. Heacock  
President and Chief Nuclear Officer  
Virginia Electric and Power Company  
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SUBJECT: SURRY POWER STATION, UNIT NOS. 1 AND 2, ISSUANCE OF AMENDMENT  
REGARDING REQUEST TO CORRECT TECHNICAL SPECIFICATION TABLE  
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A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

*/RA/*

Karen Cotton, Project Manager  
Plant Licensing Branch II-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

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GHukam, NRR/DE

ADAMS Accession No. ML100140559

\* SE transmitted by memo dated 12/9/09

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DATE	2/25/10	2/3/10	12/9/09	2/16/10	2/22/10	2/26/10

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