

Consolidated Edison Company of New York, Inc.  
4 Irving Place, New York, N.Y. 10003  
Telephone (212) 400-3819

February 2, 1972

Re Docket No. 50-247

Mr. R. C. DeYoung  
Assistant Director for PWR  
Division of Reactor Licensing  
U.S. Atomic Energy Commission  
Washington, D.C. 20545

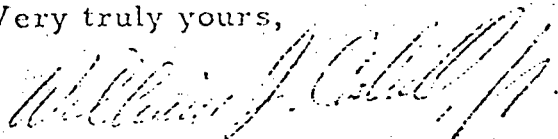
Dear Mr. DeYoung:

Your letter of January 7, 1972 asked us to provide you with our plans for conducting transient tests involving loss of reactor coolant flow; i. e., loss of power to reactor coolant pumps.

The tests already planned for Indian Point Unit #2 provide verification of the loss of coolant flow analysis presented in the Final Safety Analysis Report and demonstrate that the plant equipment, as installed, will safely withstand the consequences of loss of coolant flow. We, therefore, believe that the additional tests are unnecessary. However, in response to your requirement, we will conduct at-power loss of flow tests.

We propose simultaneous trip of two of the four reactor coolant pumps near 50% of licensed power (35 - 50%), and trip of one reactor coolant pump near full power (90 - 100%). Test procedures will be developed for this test directed toward the objective of observing system behavior during the pump coast-down. Accordingly, pertinent nuclear steam supply system parameters such as reactor coolant flows, temperatures, and pressure will be recorded and the results reviewed. These tests may be conducted concurrently with other at-power transient tests such as the turbine-generator overspeed tests. In this event, the reactor coolant pumps would be manually tripped immediately following initiation of an automatic reactor trip signal.

Very truly yours,



William J. Cahill, Jr.  
Vice President

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