



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**

REGION III  
2443 WARRENVILLE ROAD, SUITE 210  
LISLE, IL 60532-4352

January 12, 2010

Mr. Timothy J. O'Connor  
Site Vice President  
Monticello Nuclear Generating Plant  
Northern States Power Company, Minnesota  
2807 West County Road 75  
Monticello, MN 55362-9637

**SUBJECT: ERRATA FOR MONTICELLO NUCLEAR GENERATING PLANT NRC  
COMPONENT DESIGN BASES INSPECTION (CDBI) INSPECTION REPORT  
05000263/2009007(DRS)**

Dear Mr. O'Connor:

On January 6, 2010, the U.S. Nuclear Regulatory Commission (NRC) issued Inspection Report 05000263/2009007(DRS) for your Monticello Nuclear Generating Plant. The first finding and associated violation listed in the Mitigating Systems Cornerstone was inadvertently identified as a 10 CFR Part 50, Appendix B, Criterion III, "Design Control," violation.

The enclosed erratum contains the correct regulation as 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action." Please replace Page 1 of the Summary of Findings with the enclosed revised page. We regret any inconvenience this may have caused.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Should you have any questions concerning these errata, we will be pleased to discuss them with you.

Sincerely,  
*/RA/*

Ann Marie Stone, Chief  
Engineering Branch 2  
Division of Reactor Safety  
Docket No. 50-263

Docket No. 50-263  
License No. DPR-22

Enclosure: Summary of Findings Page 1  
cc w/encl: Distribution via ListServ

## SUMMARY OF FINDINGS

IR 05000263/2009007; 08/31/2009 – 12/04/2009; Monticello Nuclear Generating Plant; Component Design Bases Inspection (CDBI) and Power Uprate

The inspection was a 3-week onsite baseline inspection that focused on the design of components that are risk-significant and have low design margin; and power uprate. The inspection was conducted by regional engineering inspectors and two consultants. Five Green findings were identified by the inspectors. The findings were considered Non-Cited Violations (NCVs) of NRC regulations. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter (IMC) 0609, "Significance Determination Process" (SDP). Findings for which the SDP does not apply may be Green or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December 2006.

### A. NRC-Identified and Self-Revealed Findings

#### **Cornerstone: Barrier Integrity**

- Green. The inspectors identified an NCV of 10 CFR Part 50, Appendix B, Criterion III, "Design Control," having very low safety-significance for the failure to incorporate the actual physical configuration of the inboard main steam isolation valves (MSIVs) and the correct pneumatic system pressure drop into the pneumatic pressure requirement calculation for the inboard MSIVs. Specifically, the licensee failed to adjust the actuator moving part weight to reflect that the actuator was offset by 45 degrees instead of being mounted vertically and to correctly compute the system pressure drop. This finding was entered into the licensee's corrective action program and a preliminary calculation performed by the licensee concluded that the valves were operable.

The finding was more than minor because it was associated with the Barrier Integrity cornerstone attribute of structures, systems, components, and barrier performance, and affected the cornerstone objective of providing reasonable assurance that physical design barriers protect the public from radionuclide releases caused by accidents or events. This finding is of very low safety-significance (Green) because there was no actual barrier degradation. The inspectors did not identify a cross-cutting aspect associated with this finding because this was a legacy design issue and therefore was not reflective of current performance. (Section 1R21.3.b.(1))

#### **Cornerstone: Mitigating Systems**

Green. The inspectors identified an NCV of 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," having very low safety-significance for the failure to restore the emergency service water (ESW) piping supports to their design specifications. Specifically, although the licensee identified the existence of gaps between the ESW piping supports and the baseplates, the licensee failed to recognize that this condition did not meet seismic Category 1 design basis requirements. As a result, corrective actions were not implemented. The licensee entered this issue into its corrective action program and restored the supports to their design specifications.

Mr. Timothy J. O'Connor  
 Site Vice President  
 Monticello Nuclear Generating Plant  
 Northern States Power Company, Minnesota  
 2807 West County Road 75  
 Monticello, MN 55362-9637

SUBJECT: ERRATA MONTICELLO NUCLEAR GENERATING PLANT NRC COMPONENT  
 DESIGN BASES INSPECTION (CDBI) INSPECTION REPORT  
 05000263/2009007(DRS)

Dear Mr. O'Connor:

On January 6, 2010, the U.S. Nuclear Regulatory Commission (NRC) issued Inspection Report 05000263/2009007(DRS) for your Monticello Nuclear Generating Plant. The first finding and associated violation listed in the Mitigating Systems Cornerstone was inadvertently identified as a 10 CFR Part 50, Appendix B, Criterion III, "Design Control," violation.

The enclosed erratum contains the correct Cornerstone for Mitigating Systems as 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action." Please replace Page 1 of the Summary of Findings with the enclosed revised page. We regret any inconvenience this may have caused.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be made available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS), accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Should you have any questions concerning these errata, we will be pleased to discuss them with you.

Sincerely,  
 /RA/  
 Anne Marie Stone, Chief  
 Engineering Branch 2  
 Division of Reactor Safety

Docket No. 50-263  
 License No. DPR-22

cc w/encl: Distribution via ListServ

DOCUMENT NAME: Ltr 01\_\_10 ERRATA to Monticello 2009-007.doc

Publicly Available     Non-Publicly Available     Sensitive     Non-Sensitive  
 To receive a copy of this document, indicate in the concurrence box "C" = Copy without attach/encl "E" = Copy with attach/encl "N" = No copy

OFFICE	RIII	RIII				
NAME	AMStone for ADunlop:jb	AMStone				
DATE	01/12/10	01/12/10				

**OFFICIAL RECORD COPY**

Letter to Timothy J. O'Connor from Ann Marie Stone dated January 12, 2010

SUBJECT: ERRATA MONTICELLO NUCLEAR GENERATING PLANT NRC COMPONENT  
DESIGN BASES INSPECTION (CDBI) INSPECTION REPORT  
05000263/2009007(DRS)

DISTRIBUTION:

Susan Bagley  
RidsNrrDorLpl3-1 Resource  
RidsNrrPMMonticello  
RidsNrrDirslrib Resource  
Cynthia Pederson  
Steven Orth  
Jared Heck  
Allan Barker  
Carole Ariano  
Linda Linn  
DRPIII  
DRSIII  
Patricia Buckley  
Tammy Tomczak  
[ROPreports Resource](#)