



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

March 22, 2010

Mr. Jeffrey B. Archie
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1, ISSUANCE OF
AMENDMENT REGARDING: USE OF OPTIMIZED ZIRLO FUEL ROD
CLADDING (TAC NO: ME1489)

Dear Mr. Archie:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 182 to Renewed Facility Operating License No. NPF-12 for the Virgil C. Summer Nuclear Station, Unit No. 1, in response to your letter dated June 9, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML091620072). The amendment changes Technical Specification (TS) 5.3.1, "Fuel Assemblies," to add Optimized ZIRLO™ as an acceptable fuel rod cladding material and adds a Westinghouse topical report to the analytical methods identified in TS 6.9.1.11.

The NRC staff authorized the exemption by letter dated March 9, 2010, and published it in the *Federal Register* on March 15, 2010 (75 FR 12312).

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's next Biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink that reads "Robert Martin".

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosures:

1. Amendment No. 182 to NPF-12
2. Safety Evaluation

cc w/enclosures: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
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SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

DOCKET NO. 50-395

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 182
Renewed License No. NPF-12

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by South Carolina Electric & Gas Company (the licensee), dated June 9, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Appendix A Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-12 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 182, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This amendment is effective as of its date of issuance and shall be implemented within sixty (60) days.

FOR THE NUCLEAR REGULATORY COMMISSION



Gloria Kulesa, Chief For
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachments: Changes to the Technical
Specifications

Date of Issuance: March 22, 2010

ATTACHMENT TO LICENSE AMENDMENT NO. 182
TO RENEWED FACILITY OPERATING LICENSE NO. NPF-12
DOCKET NO. 50-395

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
<u>License</u> 3	<u>License</u> 3
<u>TS</u> 5-6 6-16 6-16a	<u>TS</u> 5-6 6-16 6-16a

- (3) SCE&G, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as amended through Amendment No. 33;
 - (4) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed neutron sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus of components; and
 - (6) SCE&G, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain, and is subject to, the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

SCE&G is authorized to operate the facility at reactor core power levels not in excess of 2900 megawatts thermal in accordance with the conditions specified herein and in Attachment 1 to this renewed license. The preoccupation tests, startup tests and other items identified in Attachment 1 to this renewed license shall be completed as specified. Attachment 1 is hereby incorporated into this renewed license.

(2) Technical Specifications and Environmental Protection Plan

The technical specifications contained in Appendix A, as revised through Amendment No. 182 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. South Carolina Electric & Gas Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

DESIGN FEATURES

5.3 REACTOR CORE

FUEL ASSEMBLIES

5.3.1 The core shall contain 157 fuel assemblies. Each fuel assembly shall consist of 264 Zircaloy-4, ZIRLO™, or Optimized ZIRLO™ clad fuel rods with an initial composition of uranium dioxide with a maximum nominal enrichment of 4.95 weight percent U-235 as fuel material. Limited substitutions of Zircaloy-4, ZIRLO™, or Optimized ZIRLO™ and/or stainless steel filler rods for fuel rods, if justified by a cycle specific reload analysis using an NRC-approved methodology, may be used. Fuel assembly configurations shall be limited to those designs that have been analyzed with applicable NRC staff-approved codes and methods, and shown by tests or cycle-specific reload analyses to comply with all fuel safety design bases. Reload fuel shall contain sufficient integral fuel burnable absorbers such that the requirements of Specifications 5.6.1.1a.2 and 5.6.1.2.b are met. A limited number of lead test assemblies that have not completed representative testing may be placed in non-limiting core locations.

CONTROL ROD ASSEMBLIES

5.3.2 The reactor core shall contain 48 full length control rod assemblies. The full length control rod assemblies shall contain a nominal 142 inches of absorber material. The nominal values of absorber material shall be 80 percent silver, 15 percent indium and 5 percent cadmium. All control rods shall be clad with stainless steel tubing.

5.4 REACTOR COOLANT SYSTEM

DESIGN PRESSURE AND TEMPERATURE

- 5.4.1 The reactor coolant system is designed and shall be maintained:
- a. In accordance with the code requirements specified in Section 5.2 of the FSAR, with allowance for normal degradation pursuant to the applicable Surveillance Requirements,
 - b. For a pressure of 2485 psig, and
 - c. For a temperature of 650°F, except for the pressurizer which is 680°F.

VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is 9914 ± 100 cubic feet at an indicated T_{avg} of 587.4°F.

5.5 METEOROLOGICAL TOWER LOCATION

5.5.1 The meteorological tower shall be located as shown on Figure 5.1-1.

ADMINISTRATIVE CONTROLS

6.9.1.9 Not used.

6.9.1.10 Not used.

CORE OPERATING LIMITS REPORT

6.9.1.11 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT prior to each reload cycle, or prior to any remaining portion of a reload cycle, for the following:

- a. Moderator Temperature Coefficient BOL and EOL Limits and 300 ppm surveillance limit for Specification 3/4.1.1.3,
- b. Shutdown Rod Insertion Limit for Specification 3/4.1.3.5,
- c. Control Rod Insertion Limits for Specification 3/4.1.3.6,
- d. Axial Flux Difference Limits, target band, and APLND for Specification 3/4.2.1,
- e. Heat Flux Hot Channel Factor, F_Q^{RTP} , $K(z)$, $W(z)$, APLND, $W(z)_{BL}$, and $F_Q(z)$ manufacturing/measurement uncertainties for Specification 3/4.2.2,
- f. Nuclear Enthalpy Rise Hot Channel Factor, $F_{\Delta H}^{RTP}$, Power Factor Multiplier, $PF_{\Delta H}$, and $F_{\Delta H}^N$ measurement uncertainties limits for Specification 3/4.2.3.

The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

- a. WCAP-9272-P-A, "WESTINGHOUSE RELOAD SAFETY EVALUATION METHODOLOGY," July 1985 (W Proprietary).

(Methodology for Specifications 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Rod Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - RCS Flow Rate and Nuclear Enthalpy Rise Hot Channel Factor.)
- b. WCAP-10216-P-A, Rev. 1A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL F_Q SURVEILLANCE TECHNICAL SPECIFICATION," February 1994 (W Proprietary).

(Methodology for Specifications 3.2.1 - Axial Flux Difference (Relaxed Axial Offset Control) and 3.2.2 - Heat Flux Hot Channel Factor (F_Q Methodology for $W(z)$ surveillance requirements).)

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT (Continued)

- c. WCAP-12945-P-A, Volume 1 (Revision 2) through Volumes 2 through 5 (Revision 1) "Code Qualification Document for Best Estimate LOCA Analysis," March 1998 (Westinghouse Proprietary).
- Liparulo, N. (W) to NRC Document Control Desk, NSD-NRC-96-4746, "Re-Analysis Work Plans Using Final Best Estimate Methodology" dated 6/13/1996.
- (Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
- d. WCAP-12472-P-A, "BEACON CORE MONITORING AND OPERATIONS SUPPORT SYSTEM," August 1994, (W Proprietary).
- (Methodology for Specifications 3.2.2 - Heat Flux Hot Channel Factor, 3.2.3 - RCS Flow Rate and Nuclear Enthalpy Rise Hot Channel Factor, and 3.2.4 - Quadrant Power Tilt Ratio.)
- e. WCAP-13749-P-A, "Safety Evaluation Supporting the Conditional Exemption of the Most Negative EOL Moderator Temperature Coefficient Measurement," March 1997, (Westinghouse Proprietary).
- (Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient.)
- f. WCAP-12610-P-A, "VANTAGE + Fuel Assembly Reference Core Report," April 1995 (W Proprietary). WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO™," July 2006 (W Proprietary).
- (Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements there to shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.



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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 182

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-12

SOUTH CAROLINA ELECTRIC & GAS COMPANY

SOUTH CAROLINA PUBLIC SERVICE AUTHORITY

VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1

DOCKET NO. 50-395

1.0 INTRODUCTION

By letter dated June 9, 2009 (Reference 1), South Carolina Electric and Gas Company (SCE&G), the licensee, submitted a license amendment request (LAR) for Technical Specification (TS) revisions for the Virgil C. Summer Nuclear Station, Unit 1 (VCSNS). The proposed change to TS 5.3.1, "Fuel Assemblies," will add Optimized ZIRLO™ as an acceptable fuel rod cladding material. SCE&G also requested an exemption from the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.46, "Acceptance Criteria for Emergency Core Cooling Systems for Light-Water Nuclear Power Reactors," and Appendix K to 10 CFR 50, "ECCS Evaluation Models," to allow the use of fuel rods clad with Optimized ZIRLO alloy for future reload applications.

The U.S. Nuclear Regulatory Commission (NRC) staff authorized the exemption by letter dated March 9, 2010, and published it in the *Federal Register* on March 15, 2010 (75 FR 12312).

The Optimized ZIRLO cladding, manufactured by Westinghouse Electric Company, is a newer version of the ZIRLO material and was approved as described in Addendum 1A of topical report WCAP-12610-P-A and CENPD-404-P-A, entitled "Optimized ZIRLO," for Westinghouse and Combustion Engineering (CE) fuel designs (Reference 3). The fuel rod burnup limits were approved to a peak rod average of 62,000 mega-watt days per metric ton uranium (MWD/MTU) for Westinghouse fuel and 60,000 MWD/MTU for CE fuel. However, the staff requires that licensees using the Optimized ZIRLO comply with the conditions and limitations listed in the safety evaluation (SE) dated June 10, 2005 (Reference 2).

2.0 REGULATORY EVALUATION

The regulations in 10 CFR 50.90, "Application for Amendment of License or Construction Permit," allow a licensee to amend or change the original license applications. 10 CFR 50.92, "Issuance of Amendment," specifies that the staff will be guided by the considerations which govern the

issuance of initial licenses to the extent applicable and appropriate in determining whether an amendment will be issued to the applicant.

The licensee requests a license amendment to add Optimized ZIRLO™ as an acceptable fuel rod cladding material in the TS. The NRC staff review of the requested amendment verifies that Optimized ZIRLO™ fuel rod cladding material is applicable to the licensee and its implementation is done in accordance with established conditions and limitations.

3.0 TECHNICAL EVALUATION

3.1 Conditions and Limitations

On June 10, 2005 (Reference 2) the NRC staff issued a safety evaluation (the SE) for Addendum 1 to topical report WCAP-12610-P-A and CENPD-404-P-A for Optimized ZIRLO™. In the SE, the staff concluded that:

Based upon demonstrated material performance in addendum 1 and in response to RAIs ... and the irradiated database, the NRC staff has approved Optimized ZIRLO™ for full batch implementation.

And in the SE conclusion the staff stated:

The NRC staff reviewed the effects of Optimized ZIRLO™ using the appropriate fuel design requirements of [Standard Review Plan] SRP 4.2 and 10 CFR Part 50, Appendix A, General Design Criteria and found that the TR provided reasonable assurance that under both normal and accident conditions, Westinghouse and CE fuel assembly designs implementing Optimized ZIRLO™ fuel cladding would be able to safely operate and comply with NRC regulations.

The NRC staff SE also stated that licensees referencing Addendum 1 to the topical report to implement Optimized ZIRLO™ must ensure compliance with ten issues as specified in the SE. SCE&G has documented its compliance with these ten conditions and limitations in its LAR (Reference 1) and has committed to ensuring compliance with them for future fuel reloads. With the exception of SE Conditions 6 and 7, the NRC staff has reviewed SCE&G's response to each of the ten SE conditions and limitations and finds each to be acceptable.

SE Conditions 6 and 7 relate to validating in-reactor performance and fuel performance models based on Lead Test Assembly (LTA) data ahead of batch application. SCE&G noted that Westinghouse has provided four references related to test data and models (references 4, 5, 6 and 7) and has provided the following statement in response to Condition 6:

LTA measured data and favorable results from visual examinations of once and twice-burned LTAs confirm, for at least two cycles of operation, that the current fuel performance models are applicable for Optimized ZIRLO™ fuel rods. Westinghouse will continue to provide additional data from the Optimized ZIRLO™ LTA programs to the NRC after new data for higher burnup/fluence become available. SCE&G will confirm that as higher burnups/fluences are achieved for Optimized ZIRLO™ clad fuel rods that the requirements of this condition will be met as it applies to VCSNS.

In response to Condition 7, SCE&G stated that the data from two cycles of operation had been evaluated and that the updated creep model had been used to predict the growth and creep in fuel rod performance. The licensee provided the favorable results to the staff. SCE&G will continue to confirm the model adequacy as higher burnups and fluences are achieved for Optimized ZIRLO™ fuel rods.

Based upon the information provided, the NRC staff finds that SCE&G's response, as discussed above, meets the requirements of SE Conditions 6 and 7 and is acceptable for the VCSNS. Therefore, the staff concludes that the Optimized ZIRLO fuel design is acceptable for use in the VCSNS to a peak rod average burnup limit of 62 GWD/MTU.

3.2 TS Revisions

3.2.1 TS Section 5.3.1 Fuel Assemblies

The licensee proposes to add Optimized ZIRLO™ in TS 5.3.1 as an acceptable fuel rod cladding material. The new sentences are stated as follows:

“... Each fuel assembly shall consist of 264 Zircaloy-4, ZIRLO™, or Optimized ZIRLO™ clad fuel rods ... Limited substitutions of Zircaloy-4, ZIRLO™, or Optimized ZIRLO™ and /or ...”

In Reference 2, the NRC staff approved Optimized ZIRLO™ fuel cladding based on (1) similarities with standard ZIRLO™, (2) demonstrated material performance, and (3) a commitment to provide irradiated data and validate fuel performance models ahead of burnups achieved in batch application. Based on the NRC staff's prior approval of Optimized ZIRLO™ fuel rod cladding, and the licensee's current and future compliance with the SE conditions and limitations, the staff concludes that this revision is acceptable for the VCSNS.

3.2.2 TS Section 6.9.1.11 Core Operating Limits Report (COLR)

SCE&G proposes to add the approved Westinghouse topical reports, WCAP-12610-P-A, “VANTAGE+ Fuel Assembly Reference Core Report,” and Addendum 1-A to WCAP-12610-P-A and CENPD-404-P-A, “Optimized ZIRLO,” to the list of references to be used in generating the Core Operating Limits Report (COLR). Based on the NRC staff's approval of those reports, the staff concludes that this revision is acceptable for the VCSNS.

Based on the NRC staff's evaluation of SCE&G's license amendment request for TS revisions the staff concludes that the Optimized ZIRLO™ fuel design is acceptable to a peak rod average burnup limit of 62 GWD/MTU and that the TS revisions are acceptable for the VCSNS.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of South Carolina State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission previously issued a proposed finding in the *Federal Register* (74 FR 64746) that the amendment involves no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1. Letter from SCE&G to NRC, "License Amendment Request For Use of Optimized Zirlo™ Fuel Rod Cladding," June 9, 2009, Agencywide Documents Access and Management System (ADAMS) Accession No. ML091620072.
2. Letter from H. N. Berkow, NRC, to J. A. Gresham, Westinghouse Electric Company, "Final Safety Evaluation for Addendum 1 to Topical Report WCAP-12610-P-A and CENPD-404-P-A, "Optimized ZIRLO™" June 10, 2005. ADAMS Accession No. ML051670403.
3. WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A, "Optimized ZIRLO™," July 2006, ADAMS Accession No. ML062080576.
4. Letter from Westinghouse to NRC, "SER Compliance with WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A 'Optimized ZIRLO™' (Proprietary)" LTR-NRC-07-1, January 4, 2007, ADAMS Accession No. ML070100389.

5. Letter from Westinghouse to NRC, "SER Compliance with WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A 'Optimized ZIRLO™'(Proprietary)" LTR-NRC-07-58, November 2007, ADAMS Accession No. ML073130562.
6. Letter from Westinghouse to NRC, "SER Compliance with WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A 'Optimized ZIRLO™' (Non-Proprietary)" LTR-NRC-07-58, Rev. 1, February 2008, ADAMS Accession No. ML080390452.
7. Letter from Westinghouse to U.S. Nuclear Regulatory Commission, "SER Compliance of WCAP-12610-P-A & CENPD-404-P-A Addendum 1-A "Optimized ZIRLO™ (Proprietary/Non-Proprietary)," LTR-NRC-08-60, December 30, 2008, ADAMS Accession No. ML090080380.

Principal contributor: S. Wu, SNPB/DSS

Date of Issuance: March 22, 2010

Mr. Jeffrey B. Archie
 Vice President, Nuclear Operations
 South Carolina Electric & Gas Company
 Virgil C. Summer Nuclear Station
 Post Office Box 88
 Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1, ISSUANCE OF AMENDMENT REGARDING: USE OF OPTIMIZED ZIRLO FUEL ROD CLADDING (TAC NO: ME1489)

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Sincerely,

/RA/

Robert E. Martin, Senior Project Manager
 Plant Licensing Branch II-1
 Division of Operating Reactor Licensing
 Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosures:

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2. Safety Evaluation

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Amendment No: ML100110377

NRR-058

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