

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

CHAPTER 14
VERIFICATION PROGRAMS

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
14.0	VERIFICATION PROGRAMS	14.1-1
14.1	SPECIFIC INFORMATION TO BE INCLUDED IN PRELIMINARY/FINAL SAFETY ANALYSIS REPORT	14.1-1
14.2	INITIAL PLANT TEST PROGRAM	14.2-1
14.2.2	Organization and Staffing	14.2-1
14.2.3	Test Procedures	14.2-2
14.2.8.1	Preoperational and/or Startup Testing for Unique or First-of-a- Kind Principal Design Features	14.2-2
14.2.8.2.1	Natural Circulation Testing	14.2-2
14.2.9	Trial Testing of Plant Operating and Emergency Procedures	14.2-3
14.2.11	Test Program Schedule	14.2-3
14.2.12	Individual Test Descriptions	14.2-3
14.2.12.1	Preoperational Tests	14.2-4
14.2.12.1.112	Personnel Monitors and Radiation Survey Instruments Preoperational Test	14.2-4
14.2.12.1.113	Ultimate Heat Sink (UHS) System Preoperational Test...	14.2-5
14.2.12.1.114	UHS ESW Pump House Ventilation System Preoperational Test	14.2-6
14.2.13	Combined License Information	14.2-7
14.3	INSPECTIONS, TESTS, ANALYSES, AND ACCEPTANCE CRITERIA	14.3-1
14.3.4.6	ITAAC for Electrical Systems	14.3-1
14.3.4.7	ITAAC for Plant Systems	14.3-1
14.3.4.10	ITAAC for Emergency Planning	14.3-2
14.3.4.12	ITAAC for Physical Security Hardware	14.3-2
14.3.6	Combined License Information	14.3-2
 APPENDIX		
APPENDIX 14A	COMPARISON OF RG 1.68 APPENDIX A VERSUS US-APWR TEST ABSTRACTS	

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

LIST OF TABLES

<u>Number</u>	<u>Title</u>
14.2-201	Comprehensive Listing of Additional Tests
14.2-202	Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

ACRONYMS AND ABBREVIATIONS

ac	alternating current
COL	Combined License
COLA	Combined License Application
CPNPP	Comanche Peak Nuclear Power Plant
CRDM	control rod drive mechanism
CVCS	chemical and volume control system
DAS	diverse actuation system
dc	direct current
DCD	Design Control Document
ECCS	emergency core cooling system
ESF	engineered safety features
ESW	essential service water
ESWS	essential service water system
FSAR	Final Safety Analysis Report
HVAC	heating, ventilation, and air conditioning
ITAAC	inspections, tests, analyses, and acceptance criteria
ITP	initial test program
LOOP	loss of offsite power
MCR	main control room
MFIV	main feedwater isolation valve
MHI	Mitsubishi Heavy Industries, Ltd
MNES	Mitsubishi Nuclear Energy Systems, Inc.
MSIV	main steam isolation valve
NDE	nondestructive examination
non-ESW	non-essential service water
NRC	U.S. Nuclear Regulatory Commission
PMWS	primary makeup water system
PWR	pressurized-water reactor
RCP	reactor coolant pump
RCS	reactor coolant system
RG	Regulatory Guide
RHRS	residual heat removal system
RTD	resistance temperature detector
SDV	safety depressurization valve
SFPCS	spent fuel pit cooling and purification system
SIS	safety injection system

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

ACRONYMS AND ABBREVIATIONS (Continued)

SORC	Station Operations Review Committee	
SSC	structure, system, and component	
UHS	ultimate heat sink	

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

14.0 VERIFICATION PROGRAMS

**14.1 SPECIFIC INFORMATION TO BE INCLUDED IN
PRELIMINARY/FINAL SAFETY ANALYSIS REPORT**

This section of the referenced Design Control Document (DCD) is incorporated by reference with no departures or supplements.

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

14.2 INITIAL PLANT TEST PROGRAM

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

14.2.2 Organization and Staffing

CP COL 14.2(2) Replace the last sentence of the second paragraph in **DCD Subsection 14.2.2** with the following.

The site-specific organization, organizational titles, organization responsibilities, and reporting relationships are consistent with US-APWR Test Program Description Technical Report, MUAP-08009 (Reference 14.2-29) with the following reconciliations.

Replace the fourth bullet in Section 3.4 of MUAP-08009 with the following.

- Mitsubishi Heavy Industries, Ltd. (MHI) and/or Mitsubishi Nuclear Energy Systems, Inc (MNES) (for preoperational testing performed on the nuclear steam support system and associated auxiliary systems)
-

Replace the second paragraph in Section 3.5 of MUAP-08009 with the following.

The test review group functions as a subcommittee of the Station Operations Review Committee (SORC) defined in Subsection 13.1.1.2.1 for initial startup testing matters. The test review group is charged with reviewing initial startup test activities and advising the SORC on the disposition of those items reviewed. The SORC may perform the test review group functions in lieu of the test review group. The primary function of the test review group is the review and approval of initial startup program test procedures, procedure revisions, and test results.

Replace the fourth bullet of the third paragraph in Section 3.5 of MUAP-08009 with the following.

- MHI and/or MNES
-

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Replace the first sentence in Section 8.2 of MUAP-08009 with the following.

Test procedures are, at a minimum, reviewed by MHI or MNES engineering, Testing, Operations, Quality Assurance, Maintenance, and Licensing.

14.2.3 Test Procedures

STD COL 14.2(12) Add the following sentence at the end of **DCD Subsection 14.2.3**.

Approved test procedures for satisfying testing requirements of Section 14.2 are made available to the NRC approximately 60 days prior to their intended use.

14.2.8.1 Preoperational and/or Startup Testing for Unique or First-of-a-Kind Principal Design Features

STD COL 14.2(11) Replace the last sentence of the second paragraph in **DCD Subsection 14.2.8.1** with the following.

First-plant-only and prototype tests are either performed in accordance with Subection 14.2.8 or a justification is provided prior to initial fuel loading that the results of the First-plant-only test and prototype test are applicable to a subsequent plant and are not required to be repeated.

14.2.8.2.1 Natural Circulation Testing

STD COL 14.2(11) Add the following text at the end of **DCD Subsection 14.2.8.2.1**.

Natural circulation test is performed in accordance with Subsection 14.2.12.2.3.9 or a justification is provided based on Subsection 14.2.8.2.1 prior to initial fuel load that the results of the US-APWR prototype test are applicable to a subsequent plant and are not required to be repeated.

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

14.2.9 Trial Testing of Plant Operating and Emergency Procedures

CP COL 14.2(7) Replace the last paragraph in **DCD Subsection 14.2.9** with the following.

A schedule for the development of plant procedures required for use during preoperational testing will be provided to the U.S. Nuclear Regulatory Commission (NRC) 12 months prior to the start of the corresponding preoperational tests. A schedule for the development of plant procedures required for use during startup testing is provided to the NRC 12 months prior to the start of fuel loading. The schedules provide sufficient detail to assure that the procedures required to support testing are available for test procedure preparation, review and performance.

14.2.11 Test Program Schedule

CP COL 14.2(7) Replace the first and second sentences of the last paragraph in **DCD Subsection 14.2.11** with the following.

An event-based schedule for conducting each major phase of the test program for the Comanche Peak Nuclear Power Plant (CPNPP) Units 3 and 4, relative to the start of fuel loading, will be provided to the NRC six months prior to the start of preoperational testing. The schedule will be periodically updated to reflect actual progress. Schedule preparation will include an assessment of overlapping test program schedules between CPNPP Units 3 and 4 and provide assurance that CPNPP Unit 3 will be given priority during the period when testing and plant staff personnel will be working on both units.

CP COL 14.2(7) Replace the third sentence of the last paragraph in **DCD Subsection 14.2.11** with the following.

Preoperational tests which satisfy inspections, tests analyses, and acceptance criteria (ITAAC) test requirements, and ITAAC test requirements which can be incorporated into preoperational tests, are correlated in **Table 14.2-202**. This correlation is used to assure that ITAAC test requirements are included in the development of preoperational testing procedures.

14.2.12 Individual Test Descriptions

Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR

CP COL 14.2(10) Replace the first sentence of the last paragraph in **DCD Subsection 14.2.12** with the following.

Testing outside the scope of the certified design is addressed in Subsections 14.2.12.1.112, 14.2.12.1.113, and 14.2.12.1.114. Additional testing for the Fire Protection System Preoperational Test is identified in Subsection 14.2.12.1.90. **Table 14.2-201** shows the comprehensive list for the new added subsections.

14.2.12.1 Preoperational Tests

STD COL 14.2(10) Add new item after item C.7 in **DCD Subsection 14.2.12.1.90** as follows.

8. Verify that local offsite fire departments utilize hose threads or adapters capable of connecting with onsite hydrants, hose couplings, and standpipe risers.

Add new subsections after **DCD Subsection 14.2.12.1.111** as follow.

STD COL 14.2(10) **14.2.12.1.112 Personnel Monitors and Radiation Survey Instruments Preoperational Test**

A. Objective

1. To demonstrate the operation, indication, and alarm functions of radiological personnel monitors and radiation survey instruments.

B. Prerequisites

1. Required construction testing is completed.
2. Test instrumentation is available and calibrated.
3. Required support systems are available.
4. Indicators, power supplies, and sensors have been calibrated as required in accordance with vendor instructions.

C. Test Method

1. Performance of each monitor and survey unit is observed and recorded during individual component tests for each unit during calibration using standard radiation sources, including verification of all alarms, annunciators, and indicators, operation of bypass,

Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR

interlock, permissive, self-test and loss of power functions, as applicable.

D. Acceptance Criterion

1. Component and, where applicable, integrated testing demonstrates that each monitor or survey unit operates as specified by vendor technical information and plant procedures, including the following, as applicable:
 - i. Alarms, annunciators, and indicators.
 - ii. Bypass, interlock, permissive, self-test, and loss of power functions.

CP COL 14.2(10) **14.2.12.1.113 Ultimate Heat Sink (UHS) System Preoperational Test**

A. Objectives

1. To demonstrate operation of the UHS cooling towers and associated fans, essential service water (ESW) pumps, and UHS transfer pumps.
2. With the basin at minimum level (end of the 30 day emergency period), to demonstrate that the ESW pumps maintain design flow rates.
3. To demonstrate the operation of the UHS transfer pumps and associated interlocks.
4. To demonstrate the operation of the UHS basin water level sensors and basin water level controls, and water chemistry monitors, controls, interlocks, and associated blowdown equipment.

B. Prerequisites

1. Required construction testing is completed.
2. Component testing and instrument calibration is completed.
3. Test instrumentation is available and calibrated.
4. Required support systems are available.
5. Required system flushing/cleaning is completed.
6. Required electrical power supplies and control circuits are energized and operational.
7. Makeup water to the UHS basins is available.

Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR

C. Test Method

1. System component control and interlock circuits and alarms are verified, including cooling tower fan logic, basin water level sensors, makeup water control, basin process chemical sensors, blowdown control valves, and UHS transfer pump interlocks.
2. The performance of each ESW pump is monitored as basin water level is decreased to the minimum water level (end of the 30 day emergency period).
3. Basin water level and chemistry controls are monitored during continuous operations in the water level and chemistry control mode.

D. Acceptance Criteria

1. With the basin at minimum level (end of the 30 day emergency period), each ESW pump maintains design flow rates.
2. UHS transfer pumps and associated interlocks operate as discussed in Subsection 9.2.5.
3. UHS basin water level sensors and basin water level controls, and water chemistry monitors, controls, interlocks and associated blowdown equipment operate as discussed in Subsection 9.2.5.

CP COL 14.2(10) **14.2.12.1.114 UHS ESW Pump House Ventilation System Preoperational Test**

A. Objectives

1. To demonstrate operation of the UHS ESW pump house ventilation system.

B. Prerequisites

1. Required construction testing is completed.
2. Component testing and instrument calibration are completed.
3. Test instrumentation is available and calibrated.
4. Required support systems are available.

C. Test Method

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

1. Simulate interlock signals for each exhaust fan and unit heater and verify operation and annunciation.
2. Verify that alarms and status indications are functional.
3. Verify design airflow.

D. Acceptance Criteria

1. UHS ESW pump house ventilation system operates on the proper signal (see **Subsection 9.4.5**).
2. All alarms annunciate properly.

14.2.13 Combined License Information

Replace the content of **DCD Subsection 14.2.13** with the following.

14.2(1) Deleted from the DCD

CP COL 14.2(2) **14.2(2)** Organization and staffing

This COL item is addressed in **Subsection 14.2.2**.

14.2(3) Deleted from the DCD.

14.2(4) Deleted from the DCD.

14.2(5) Deleted from the DCD.

14.2(6) Deleted from the DCD.

CP COL 14.2(7) **14.2(7)** Initial test program schedule and cross-reference of test abstracts with ITAAC

This COL item is addressed in Subsections **14.2.9**, **14.2.11** and **Table 14.2-202**

14.2(8) Deleted from the DCD.

14.2(9) Deleted from the DCD.

CP COL 14.2(10)
STD COL 14.2(10) **14.2(10)** Site-specific test abstracts

This COL item is addressed in Subsections **14.2.12.1.90.C.8**, **14.2.12.1.112**, **14.2.12.1.113**, and **14.2.12.1.114**, **Table 14.2-201**, and **Appendix 14A**.

Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR

STD COL 14.2(11) **14.2(11)** *First-plant only test and prototype test*

This COL item is addressed in Subsections 14.2.8.1 and 14.2.8.2.1.

STD COL 14.2(12) **14.2(12)** *Approved Test procedures*

This COL item is addressed in Subsection 14.2.3

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14.2-201

Comprehensive Listing of Additional Tests

	Section	Test
STD COL 14.2(10)	14.2.12.1.90.C.8	Local Fire Department Hose Thread Compatibility Test
STD COL 14.2(10)	14.2.12.1.112	Personnel Monitors and Radiation Survey Instruments Preoperational Test
CP COL 14.2(10)	14.2.12.1.113	Ultimate Heat Sink (UHS) Preoperational Test
CP COL 14.2(10)	14.2.12.1.114	UHS ESW Pump House Ventilation System Preoperational Test

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

CP COL 14.2(7)

Table 14.2-202 (Sheet 1 of 6)

Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

Test Description	Tier 2 Section	Tier 1 Section
Reactor coolant system (RCS) Hot Functional	14.2.12.1.1	2.4.2
Pressurizer Pressure and Water Level Control	14.2.12.1.2	-
Reactor coolant pump (RCP) Initial Operation	14.2.12.1.3	2.4.2
Pressurizer Safety Depressurization Valve (SDV)	14.2.12.1.4	2.4.2
Pressurizer Relief Tank	14.2.12.1.5	-
RCS	14.2.12.1.6	2.4.2
Reactor Internals Vibration	14.2.12.1.7	2.4.1
RCS Cold Hydrostatic	14.2.12.1.8	2.2, 2.4.1, 2.4.2
Reactor Control, Rod Control, and Rod Position Indication	14.2.12.1.9	(2.5.5)
Control rod drive mechanism (CRDM) Motor Generator Set	14.2.12.1.10	-
CRDM Initial Timing	14.2.12.1.11	-
Chemical and Volume Control System (CVCS) – Boric Acid Blending	14.2.12.1.12	2.4.6
CVCS – Charging and Seal Water	14.2.12.1.13	2.4.6
CVCS – Letdown	14.2.12.1.14	2.4.6
RCS Lithium Addition and Distribution	14.2.12.1.15	-
Primary Makeup Water System (PMWS)	14.2.12.1.16	(2.7.6.11)
Reactor Trip System and engineered safety features (ESF) System Response Time	14.2.12.1.17	-
Reactor Trip System and ESF System Logic	14.2.12.1.18	2.5.1, 2.7.1.1
Resistance Temperature Detectors (RTDs)/Thermocouple Cross-Calibration	14.2.12.1.19	-
Diverse Actuation System (DAS) Actuation	14.2.12.1.20	2.5.3
Main Steam Supply System	14.2.12.1.21	2.7.1.2, (2.7.1.6)
Residual Heat Removal System (RHRS)	14.2.12.1.22	2.4.5

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14.2-202 (Sheet 2 of 6)

Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

Test Description	Tier 2 Section	Tier 1 Section
Main Steam Isolation Valve (MSIV), Main Feedwater Isolation Valve (MFIV), and Main Steam Check Valve	14.2.12.1.23	2.7.1.2, 2.7.1.9
Motor-Driven Emergency Feedwater System	14.2.12.1.24	2.7.1.11
Turbine-Driven Emergency Feedwater System	14.2.12.1.25	2.7.1.11
Extraction Steam	14.2.12.1.26	(2.7.1.1)
Main Turbine System Valves	14.2.12.1.27	2.7.1.1
Condensate System	14.2.12.1.28	(2.7.1.9)
Feedwater System	14.2.12.1.29	2.7.1.9
Feedwater Heater and Drain Systems	14.2.12.1.30	(2.7.1.9)
Condensate Polishing System	14.2.12.1.31	(2.7.1.8)
Main Condenser Evacuation System	14.2.12.1.32	(2.7.1.4)
Circulating Water System	14.2.12.1.33	(2.7.1.7)
Essential Service Water System (ESWS)	14.2.12.1.34	2.7.3.1
Main and Unit Auxiliary Transformers	14.2.12.1.35	2.6.1
Reserve Auxiliary Transformers	14.2.12.1.36	(2.6.1)
Non-Class 1E Alternating Current (ac) Distribution	14.2.12.1.37	(2.6.1)
6.9 kV Class 1E System	14.2.12.1.38	2.6.1
480 V Class 1E Switchgear	14.2.12.1.39	2.6.1
480 V Class 1E Motor Control Center	14.2.12.1.40	2.6.1
120 V ac Class 1E	14.2.12.1.41	2.5.1, 2.6.3
Emergency Lighting System	14.2.12.1.42	2.6.6
Normal Lighting System	14.2.12.1.43	(2.6.6)
Class 1E Gas Turbine Generator	14.2.12.1.44	2.6.4

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14.2-202 (Sheet 3 of 6)

Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

Test Description	Tier 2 Section	Tier 1 Section
Class 1E Bus Load Sequence	14.2.12.1.45	2.4.1, 2.4.2, 2.4.4, 2.4.5, 2.4.6, 2.6.1, 2.6.3, 2.6.4, 2.7.1.2, 2.7.1.9, 2.7.1.10, 2.7.1.11, 2.7.3.1, 2.7.3.3, 2.7.3.5, 2.7.5.1, 2.7.5.2, 2.7.5.4, 2.7.6.3, 2.7.6.6, 2.7.6.7, 2.7.6.13, 2.11.2, 2.11.3
Alternate ac Power Sources for Station Black Out	14.2.12.1.46	2.6.5
125 V Direct Current (dc) Class 1E	14.2.12.1.47	2.4.1, 2.4.2, 2.4.4, 2.4.5, 2.4.6, 2.6.2, 2.6.3, 2.7.1.2, 2.7.1.9, 2.7.1.10, 2.7.1.11, 2.7.3.1, 2.7.3.3, 2.7.3.5, 2.7.5.1, 2.7.5.2, 2.7.5.4, 2.7.6.3, 2.7.6.6, 2.7.6.7, 2.7.6.13, 2.11.2, 2.11.3
125 V DC Class 1E Minimum Load Voltage Verification	14.2.12.1.48	-
125 V DC non-Class 1E	14.2.12.1.49	-
Dynamic State Vibration Monitoring of Safety Related and High-Energy Piping	14.2.12.1.50	-
Steady State Vibration Monitoring of Safety Related and High-Energy Piping	14.2.12.1.51	-
Thermal Expansion Testing	14.2.12.1.52	-

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14.2-202 (Sheet 4 of 6)

Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

Test Description	Tier 2 Section	Tier 1 Section
Class 1E Gas Turbine Generator Sequence – Loss of Offsite Power (LOOP) Sequence and LOOP Sequence with Emergency Core Cooling System (ECCS) Actuation Signal	14.2.12.1.53	2.6.4
Safety Injection System (SIS)	14.2.12.1.54	2.4.4
ECCS Actuation and Containment Isolation Logic	14.2.12.1.55	2.4.4, 2.11.2
Safety Injection Check Valve	14.2.12.1.56	2.4.4
Safety Injection Accumulator	14.2.12.1.57	2.4.4
Containment Spray System	14.2.12.1.58	2.11.3
Refueling Water Storage System	14.2.12.1.59	-
Essential Chilled Water System	14.2.12.1.60	2.7.3.5
Containment Structural Integrity	14.2.12.1.61	2.2
Containment Local Leakrate	14.2.12.1.62	2.2, 2.11.2
Containment Integrated Leak Rate	14.2.12.1.63	2.2
Containment Hydrogen Monitoring and Control System	14.2.12.1.64	2.11.4
CRDM Cooling System	14.2.12.1.65	(2.7.5.3)
Reactor Cavity Cooling System	14.2.12.1.66	(2.7.5.3)
Containment High Volume Purge System	14.2.12.1.67	2.8, (2.7.5.3)
Containment Low Volume Purge System	14.2.12.1.68	2.8, (2.7.5.3)
Containment Fan Cooler System	14.2.12.1.69	(2.7.5.3)
Annulus Emergency Exhaust System	14.2.12.1.70	2.7.5.2
RCS Leak Rate	14.2.12.1.71	-
Loose Parts Monitoring System	14.2.12.1.72	(2.4.3)
Seismic Monitoring System	14.2.12.1.73	-
Incore Instrumentation System	14.2.12.1.74	(2.5.5)
Nuclear Instrumentation System	14.2.12.1.75	(2.5.5)
Remote Shutdown	14.2.12.1.76	2.5.2
Miscellaneous Leakage Detection System	14.2.12.1.77	(2.7.6.8)

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14.2-202 (Sheet 5 of 6)

Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

Test Description	Tier 2 Section	Tier 1 Section
Process and Effluent Radiological Monitoring System, Area Radiation Monitoring System and Airborne Radioactivity Monitoring System	14.2.12.1.78	(2.5.5), (2.7.6.6), (2.7.6.13)
High-Efficiency Particulate Air Filters and Charcoal Absorbers	14.2.12.1.79	2.7.5.1, 2.7.5.2
Liquid Waste Management System	14.2.12.1.80	2.4.7, 2.7.4.1, (2.7.6.8)
Gaseous Waste Management System	14.2.12.1.81	2.7.4.2
Solid Waste Management System	14.2.12.1.82	(2.7.4.3)
Steam Generator Blowdown System	14.2.12.1.83	2.7.1.10
Sampling System	14.2.12.1.84	2.7.6.7
Spent Fuel Pit Cooling and Purification System (SFPCS)	14.2.12.1.85	2.7.6.3
Fuel Handling System	14.2.12.1.86	2.7.6.4
Component Cooling Water System	14.2.12.1.87	2.7.3.3
Turbine Component Cooling Water System	14.2.12.1.88	(2.7.3.4)
Secondary Side Chemical Injection System	14.2.12.1.89	(2.7.1.12)
Fire Protection System	14.2.12.1.90	2.7.6.9
Instrument Air System	14.2.12.1.91	(2.7.2)
Station Service Air System	14.2.12.1.92	(2.7.2)
Boron Recycle System	14.2.12.1.93	-
Offsite Communication System	14.2.12.1.94	2.7.6.10
Inplant Communication System	14.2.12.1.95	2.7.6.10
Safeguard Component Area Heating, Ventilation, and Air Conditioning (HVAC) System	14.2.12.1.96	2.7.5.2
Emergency Feedwater Pump Area HVAC System	14.2.12.1.97	2.7.5.2
Class 1E Electrical Room HVAC System	14.2.12.1.98	2.7.5.2
Auxiliary Building HVAC System	14.2.12.1.99	2.7.5.4, 2.8
Main Steam/Feedwater Piping Area HVAC System	14.2.12.1.100	2.7.5.4
Main Control Room (MCR) HVAC System (including MCR Habitability)	14.2.12.1.101	2.7.5.1
Non-Class 1E Electrical Room HVAC System	14.2.12.1.102	2.7.5.4

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14.2-202 (Sheet 6 of 6)

Comparison of Tier 2 Preoperational Tests with Tier 1 Test Requirements

Test Description	Tier 2 Section	Tier 1 Section
Technical Support Center HVAC System	14.2.12.1.103	2.7.5.4
Non-Essential Chilled Water System	14.2.12.1.104	2.7.3.6
Vessel Servicing	14.2.12.1.105	2.7.6.5
Safety-Related Component Area HVAC System	14.2.12.1.106	2.7.5.2
Pressurizer Heater and Spray Capability and Continuous Spray Flow Verification	14.2.12.1.107	-
Non-Essential Service Water (non-ESW) System	14.2.12.1.108	(2.7.3.2)
Condensate Storage Facilities System	14.2.12.1.108	(2.7.6.11)
Turbine Building Area Ventilation System (General Mechanical Area)	14.2.12.1.110	(2.7.5.5)
Turbine Building Area Ventilation System (Electric Equipment Area)	14.2.12.1.111	(2.7.5.5)
RCPB Leak Detection Systems Preoperational Test	14.2.12.1.115	2.4.7
Equipment and Floor Drainage System Preoperational Test	14.2.12.1.116	2.7.6.8
Compressed Gas System Preoperational Test	14.2.12.1.117	(2.7.2)

Note: Tier 1 sections in parentheses indicate inspection activities.

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

**14.3 INSPECTIONS, TESTS, ANALYSES, AND ACCEPTANCE
CRITERIA**

This section of the referenced DCD is incorporated by reference with the following departures and/or supplements.

14.3.4.6 ITAAC for Electrical Systems

STD COL 14.3(1) Add the following paragraph as the last paragraph in **DCD Subsection 14.3.4.6**.

The ITAAC for the site-specific interfaces in the electrical systems are developed to correspond to Section 3.2 of Tier 1 of the referenced DCD. The site-specific interfaces are the offsite power system and the ITAAC for the interface requirement with the offsite power system is provided in **Part 10** of the Combined License Application (COLA).

14.3.4.7 ITAAC for Plant Systems

CP COL 14.3(1) Replace the last paragraph in **DCD Subsection 14.3.4.7** with the following.

The selection criteria and methodology provided in Section 14.3 of the referenced DCD are utilized as the site-specific selection criteria and methodology for ITAAC for site-specific systems. In general, the ITAAC for site-specific systems are developed to correspond to the interface requirements in Tier 1 of the referenced DCD. For those site-specific systems that do not have a safety function sufficiently significant to meet the selection criteria for ITAAC, the system is identified with the designation "No entry for this system". ITAAC for the site-specific portion of the plant systems are provided in Part 10 of the Combined License Application (COLA). There are only two site-specific systems, the UHS system and ESWS (portions of the outside scope of the certified design) including the site-specific structures, and the UHS ESWS pump house ventilation system, which are addressed in **Part 10** of the COLA.

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

14.3.4.10 ITAAC for Emergency Planning

STD COL 14.3(2) Replace the last paragraph in **DCD Subsection 14.3.4.10** with the following.

The selection criteria and methodology provided in Section 14.3 of the referenced DCD are utilized as the site-specific selection criteria and methodology for the facility's emergency planning ITAAC. The ITAAC conform to the guidance in this subsection, as modified to reflect the design and specific emergency planning program requirements. The ITAAC for the facility's emergency planning are provided in **Part 10** of the COLA.

14.3.4.12 ITAAC for Physical Security Hardware

STD COL 14.3(3) Replace the last paragraph in **DCD Subsection 14.3.4.12** with the following.

The selection criteria and methodology provided in Section 14.3 of the referenced DCD are utilized as the site-specific selection criteria and methodology for site-specific physical security hardware ITAAC not addressed in the DCD. The ITAAC conform to the guidance in this subsection and are consistent with the applicable generic physical security ITAAC in SRP 14.3.12 (Reference 14.3-16) developed by the NRC in coordination with the Nuclear Energy Institute. The site-specific physical security hardware ITAAC are provided in **Part 10** of the COLA.

14.3.6 Combined License Information

Replace the content of **DCD Subsection 14.3.6** with the following.

STD COL 14.3(1) **14.3(1)** *ITAAC for site-specific systems*

CP COL 14.3(1)

This COL item is addressed in Subsections 14.3.4.6 and 14.3.4.7

STD COL 14.3(2) **14.3(2)** *ITAAC for emergency planning*

This COL item is addressed in Subsection 14.3.4.10.

STD COL 14.3(3) **14.3(3)** *ITAAC For Physical Security Hardware*

This COL item is addressed in Subsection 14.3.4.12

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

APPENDIX 14A

**COMPARISON OF RG 1.68 APPENDIX A VERSUS
US-APWR TEST ABSTRACTS**

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

APPENDIX 14A

COMPARISON OF RG 1.68 APPENDIX A VERSUS
US-APWR TEST ABSTRACTS

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
Appendix 14A	COMPARISON OF RG 1.68 APPENDIX A VERSUS US-APWR TEST ABSTRACTS	14A-1

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

LIST OF TABLES

<u>Number</u>	<u>Title</u>
14A-201	Conformance Matrix of RG 1.68 Appendix A Guidance versus Added Test Abstracts in the FSAR

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

**APPENDIX 14A COMPARISON OF RG 1.68 APPENDIX A VERSUS
US-APWR TEST ABSTRACTS**

This appendix of the referenced DCD is incorporated by reference with the following departures and/or supplements.

CP COL (10) Add the following text after the last sentence.

The added test abstracts in the Final Safety Analysis Report (FSAR) are correlated to Regulatory Guide (RG) 1.68 Appendix A in Table 14A-201.

**Comanche Peak Nuclear Power Plant, Units 3 & 4
COL Application
Part 2, FSAR**

Table 14A-201

**Conformance Matrix of RG 1.68 Appendix A Guidance versus
Added Test Abstracts in the FSAR**

	RG 1.68 Appendix A	Section Number	Typical Test
CP COL(10)	1.h.(7)	14.2.12.1.114	UHS ESW Pump House Ventilation System Preoperational Test
CP COL(10)	1.h.(10)	14.2.12.1.113	Ultimate Heat Sink (UHS) Preoperational Test
STD COL(10)	1.k.(2)	14.2.12.1.112	Personnel Monitors and Radiation Survey Instruments
CP COL(10)	1.n.(14) (a)	14.2.12.1.114	UHS ESW Pump House Ventilation System Preoperational Test