

January 8, 2010

SAFEGUARDS INFORMATION

Enclosure Contains Safeguards Information.
When separated from enclosure, this
Document is not Safeguards Information.

Mr. Timothy S. Rausch
Senior Vice President and Chief Nuclear Officer
PPL Susquehanna, LLC
769 Salem Blvd
Berwick, PA 18603-0467

SUBJECT: SUSQUEHANNA STEAM ELECTRIC STATION, UNITS 1 AND 2 -
SUPPLEMENTAL INSPECTION FOR A WHITE FINDING, INSPECTION
REPORT NOS. 05000387/2009406 AND 05000388/2009406

Dear Mr. Rausch:

On December 16, 2009, the NRC completed a supplemental inspection pursuant to Inspection Procedure 95001 at your Susquehanna Steam Electric Station to follow up on a previous security inspection finding. The enclosed report documents the inspection results, which were discussed at the exit meeting on December 16, 2009, with you and other members of your staff. The NRC was informed of your readiness for the inspection on December 3, 2009.

The inspection examined activities conducted under your licenses as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your licenses. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel.

Based on the results of this inspection, no findings of significance were identified.

Warning: Violation of Section 147 of the Atomic Energy Act, "Safeguards Information" is subject to civil and criminal penalties	SAFEGUARDS INFORMATION DETERMINATION MADE BY: NAME/TITLE: <u>James M. Trapp, Chief</u> ORGANIZATION: <u>RI/DRS/PSB1</u> BASIS: - DG-SGI-1 (300) Physical Protection Program Signature <u> /RA/ </u> DATE: 01/08/10
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T. Rausch

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In accordance with 10 CFR 2.390(b) (1) (ii), the NRC is waiving the affidavit requirements for your response, if any. This practice will ensure that your response will not be made available electronically for public inspection in the NRC Public Document Room or from the NRC's document system, ADAMS. If Safeguards Information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.22. Otherwise, mark your entire response "Security-Related Information - Withhold Under 10 CFR 2.390" and follow the instructions for withholding in 10 CFR 2.390(b) (1).

Sincerely,

/RA/

James M. Trapp, Chief
Plant Support Branch 1
Division of Reactor Safety

Docket Nos. 50-387, 50-388
License Nos. NPF-14, NPF-22

Enclosure: Inspection Report Nos. 05000387/2009406 and 05000388/2009406
w/Attachment: Supplemental Information (**SGI**)

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/RA/

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Plant Support Branch 1
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