

January 5, 2010

Mr. Scott Head, Manager
Regulatory Affairs
STP Nuclear Operating Company
P. O. Box 289
Wadsworth, TX 77483

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION LETTER NO. 007 RELATED TO
SRP SECTION 05.02.02 FOR THE ABWR DESIGN CERTIFICATION RULE
AMENDMENT APPLICATION

Dear Mr. Head:

By letter dated June 20, 2009, STP Nuclear Operating Company (STPNOC) submitted for approval an application to amend the ABWR design certification rule (DCR) pursuant to 10 CFR Part 52. The U. S. Nuclear Regulatory Commission (NRC) staff is performing a detailed review of this application to enable the staff to reach a conclusion on the safety of the proposed application.

The NRC staff has identified that additional information is needed to continue portions of the review. The staff's request for additional information (RAI) is contained in the enclosure to this letter.

To support the review schedule, you are requested to respond within **30** days of the date of this letter. If changes are needed to the safety analysis report, the staff requests that the RAI response include the proposed wording changes.

S. Head

-2-

If you have any questions or comments concerning this matter, I can be reached at 301-415-8484 or by e-mail at Tom.Tai@nrc.gov.

Sincerely,

/RA/

Tom M. Tai, Senior Project Manager
ABWR Projects Branch
Division of New Reactor Licensing
Office of New Reactors

Docket Nos. 52-001

eRAI Tracking No. 4157

Enclosures:
Request for Additional Information

cc: William Mookhoek
Fred Puleo
Coley Chappell

S. Head

-2-

If you have any questions or comments concerning this matter, I can be reached at 301-415-8484 or by e-mail at Tom.Tai@nrc.gov.

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|--------|----------|-----------|---------|
| OFFICE | SRSB/TR | SRSB/BC | NGE2/PM |
| NAME | GThomas | JDonoghue | TTai |
| DATE | 12/14/09 | 1/4/10 | 1/5/10 |

***Approval captured electronically in the electronic RAI system.**

OFFICIAL RECORD COPY

Request for Additional Information No. 4157 Revision 5

**ABWR Design Certification Amendment Project
South Texas Project Nuclear Operating Co**

Docket No. 52-001

SRP Section: 05.02.02 - Overpressure Protection

Application Section: DCD Figure 5.1-3

QUESTIONS for Reactor System, Nuclear Performance and Code Review (SRSB)

05.02.02-3

DCD Figure 5.1-3 (Sheet 4 of 11) Describe the type of analyses performed to ensure that the non-safety AFI line connected to the CUW tie-in lines to the feedwater system is designed such that it will not inadvertently impact the ability of safety-related RCIC system which injects to the reactor through the Feedwater system to perform their intended function. Also, summarize the results of these analyses.