

5.0 DOSE MODELING

PURPOSE OF THIS SECTION

The purpose of this section is to describe dose modeling performed for Phase 1 of the decommissioning to establish cleanup criteria that will not limit options for Phase 2 of the decommissioning.

INFORMATION IN THIS SECTION

This section provides the following information:

- Section 5.1 contains introductory material to place information in the following sections into context.
- Section 5.2 describes the **base-case and alternative** conceptual models and the mathematical model (RESRAD) used to develop derived concentration guideline levels (DCGLs) for 18 radionuclides of interest in surface soil, subsurface soil, and streambed sediment. It identifies the results in terms of DCGL_w values. **It discusses the deterministic** sensitivity analyses of model input parameters. **It also describes the probabilistic uncertainty analysis and the multi-source model for subsurface soil DCGLs that was found to be limiting for many radionuclides of interest.**
- Section 5.3 discusses considerations related to dose integration and describes analyses performed to ensure that cleanup criteria used in Phase 1 will not limit Phase 2 decommissioning options.
- Section 5.4 provides cleanup goals; describes the process for refining the DCGLs and these cleanup goals; addresses use of a surrogate radionuclide in field measurements; provides preliminary, order-of-magnitude dose assessments related to remediation of subsurface soil; and provides for final dose assessments after completion of the Phase 1 final status surveys.

RELATIONSHIP TO OTHER PLAN SECTIONS

To put into perspective the information in this section, one must consider:

- The information in Section 1 on the project background and those facilities and areas within the scope of this plan,
- The facility descriptions in Section 3,
- The information on site radioactivity in Section 4,
- The information in Section 6 on the as low as reasonably achievable (ALARA) analysis,
- The information in Section 9 on **radiation surveys**,
- The information in Appendix C that supplements the content of this section,
- The information in Appendix D on engineered barriers and groundwater flow fields, and
- **The information in Appendix E on details of the probabilistic uncertainty analysis.**

5.1 Introduction

To help place the dose modeling into context, it is useful to consider information about the applicable requirements and guidance, information on the environmental media of interest, and information relevant to consideration of doses from different parts of the project premises, along with information on matters that could impact dose modeling such as long-term erosion and potential changes in groundwater flow.

5.1.1 Applicable Requirements and Guidance

As explained in Section 1, certain areas of the project premises are being remediated in Phase 1 of the decommissioning to NRC's unrestricted release criteria in 10 CFR 20.1402. These criteria state that a site will be considered acceptable for unrestricted use if the residual radioactivity that is distinguishable from background radiation results in a total effective dose equivalent to an average member of the critical group that does not exceed 25 mrem per year, including that from groundwater sources of drinking water, and the residual radioactivity has been reduced to levels that are ALARA.

NRC provides guidance (NRC 2006) on two approaches that may be used to determine that these unrestricted release criteria have been achieved:

- (1) The dose modeling approach, which involves characterizing the site – after remediation, if necessary – and performing a dose assessment; and
- (2) The DCGL and final status survey approach, which involves developing or using DCGLs and performing a final status survey to demonstrate that the DCGLs have been met.

NRC observes that the second option is usually the more efficient or simpler method and that these two approaches are not mutually exclusive; they are just different approaches to show that the potential dose from a remediated site is acceptable (NRC 2006).

As explained below, DOE is using the DCGL approach in Phase 1 of the decommissioning and then, after remediation of subsurface soil in the two **major** areas of interest, will perform dose modeling using Phase 1 final status survey data to estimate potential future doses from these areas assuming the rest of the project premises were to also be cleaned up to the unrestricted release criteria in 10 CFR 20.1402.

DCGLs and Cleanup Goals

DCGLs are radionuclide-specific concentration limits used during decommissioning to achieve the regulatory dose standard that permit the release of the property and termination of the license. The DCGL applicable to the average concentration over a survey unit is called the DCGL_W and the DCGL applicable to limited areas of elevated concentrations within a survey unit is called the DCGL_{EMC} (NRC 2006). However, Phase 1 of the decommissioning will not result in the release of any property or in termination of the NRC license for the site. As explained below, cleanup goals below the DCGLs are used to ensure that Phase 1 criteria do not limit Phase 2 options.

5.1.2 Context for DCGL Development

Figure 5-1 shows the areas of interest for surface soil, subsurface soil, and streambed sediment for which separate DCGLs have been developed. **Each area** is discussed below.

WVDP PHASE 1 DECOMMISSIONING PLAN

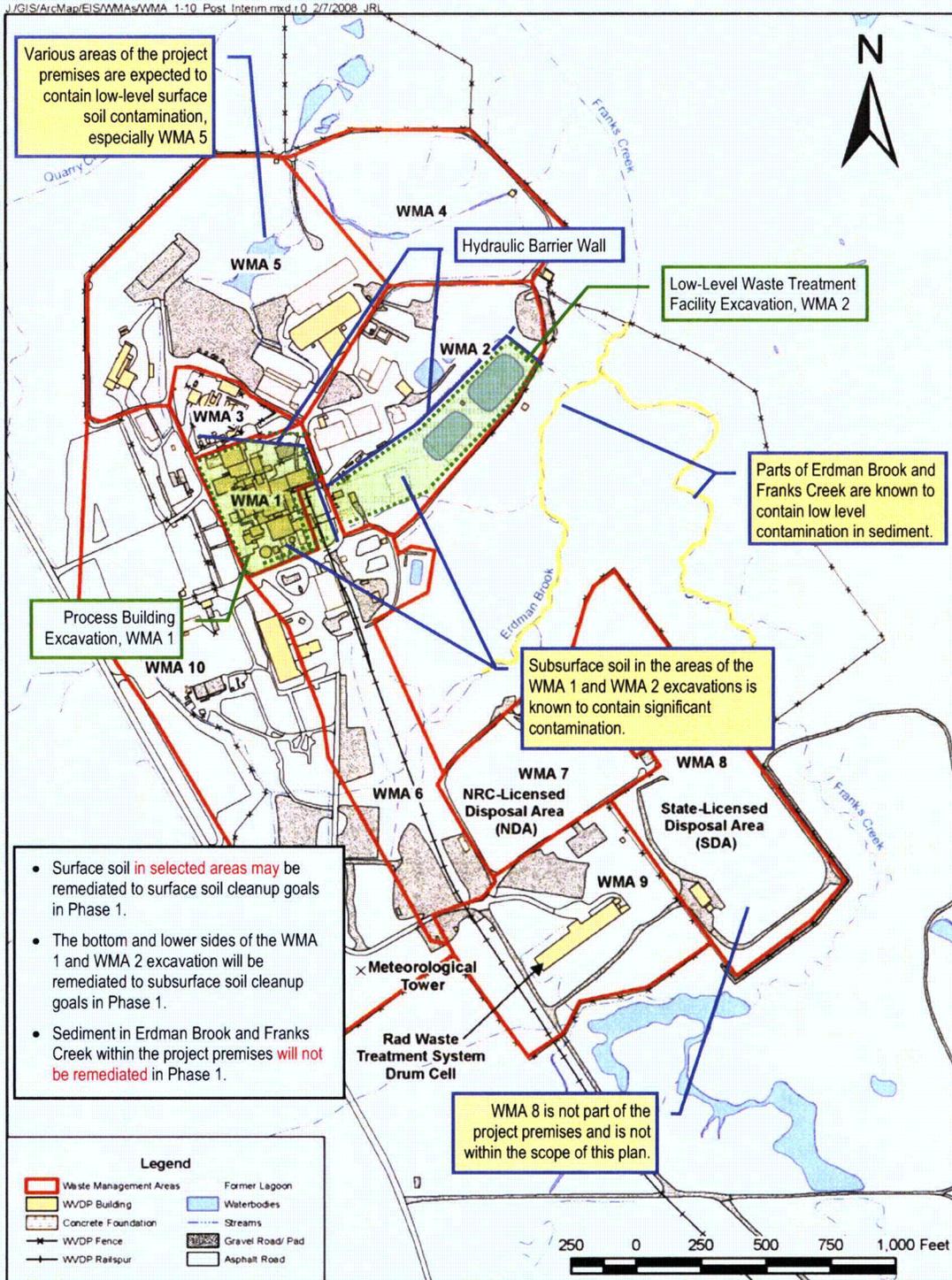


Figure 5-1. Areas of Interest – Surface Soil, Subsurface Soil, and Streambed Sediment Within the Project Premises

Surface Soil

As explained in Section 1 of this plan, surface soil and sediment in drainage ditches on the project premises will be characterized for **radioactivity** to better define the nature and extent of radioactive contamination. Section 4.2 summarizes available data on radioactivity in these environmental media. Available data indicate that radioactive contamination is present in some areas but the magnitude and areal extent of this contamination have not been fully defined. Figure 4-6 shows locations where soil and sediment **are** known to have radioactivity concentrations in excess of background.

Cs-137 concentrations in excess of background have been measured in surface soil samples from all waste management areas (WMAs) where samples have been collected, with the highest measured concentration being 280 pCi/g. Sr-90 concentrations above background have been measured in surface soil samples from several WMAs, with a maximum of 12 pCi/g. Data on other radionuclides in surface soil are very limited, but above-background concentrations of Pu-238, Pu-239/240, and Am-241 have been identified as indicated in Section 4.2.

DCGLs for surface soil based on the unrestricted **release** criteria in 10 CFR 20.1402 serve two purposes:

- They will support remediation of surface soil on selected portions of the project premises in Phase 1 of the **decommissioning, and**
- They will support decision-making for Phase 2 of the decommissioning.

The surface soil DCGLs and cleanup goals apply only to areas where there is no subsurface contamination, i.e., contamination below a depth of one meter.

Subsurface Soil

The subsurface soil DCGLs, which are also based on the unrestricted release criteria of 10 CFR 20.1402, apply only to the bottoms and lower sides of the two large excavations to be dug to remove facilities in WMA 1 and WMA 2.¹ Figure 5-2 shows a conceptual cross section view of the planned WMA 1 excavation with representative data on Sr-90 concentrations. Figure 5-3 shows a conceptual cross section view of the planned WMA 2 excavation with representative data. Both excavations will extend one foot or more into the Lavery till, as indicated in Section 7.

As explained in Section 1 and detailed in Section 7, the Process Building and the other facilities in WMA 1 will be completely removed during Phase 1 of the decommissioning, along with the source area of the north plateau groundwater plume. The excavation for this purpose will be approximately 2.8 acres in size and extend more than 40 feet below the ground into **the unweathered** Lavery till. Figure 5-1 shows the approximate location of this excavation.

¹ The subsurface soil DCGLs will be applied to the sides of these excavations at depths greater than three feet below the surface; the surface soil DCGLs would be applied to the portions of the excavation sides closer to the ground surface. Note that the sides of the excavations that are upgradient or cross-gradient (i.e., not hydraulically downgradient) of the contamination source are not expected to be contaminated.

These DCGLs may also be applicable to excavations made in Phase 2 of the decommissioning depending on the approach selected for Phase 2 and other factors if the conceptual models **are** representative of the Phase 2 conditions.

WVDP PHASE 1 DECOMMISSIONING PLAN

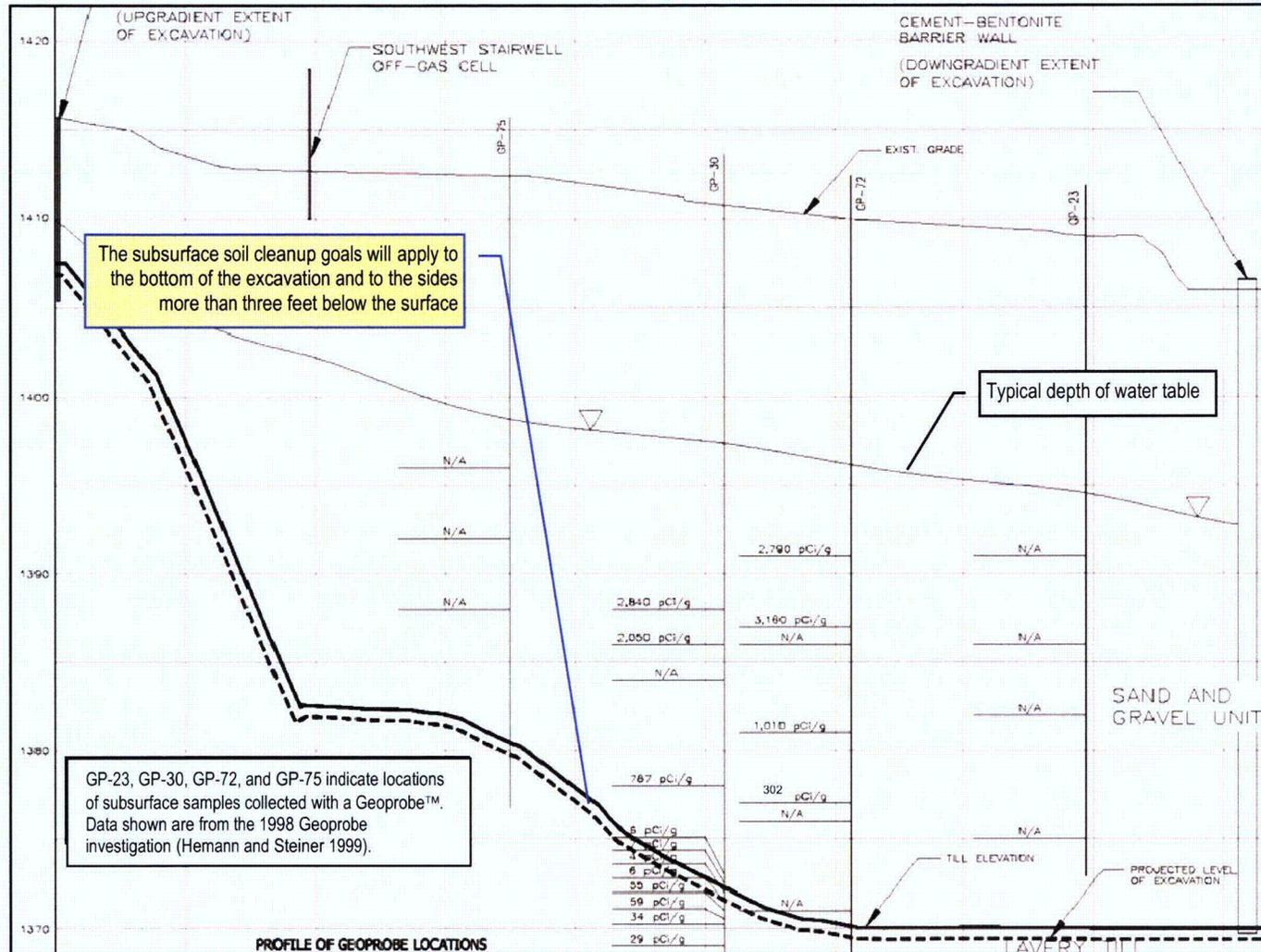


Figure 5-2. Conceptual Cross Section View of WMA 1 Excavation With Representative Soil Data on Sr-90 Concentrations (See Section 4.2 for more data and Section 7 for the excavation details.)

WVDP PHASE 1 DECOMMISSIONING PLAN

J:/GIS/ArcMap/EIS/Lagoon_Cross_Section_A_1.mxd 2/04/2009 JRL/fjc

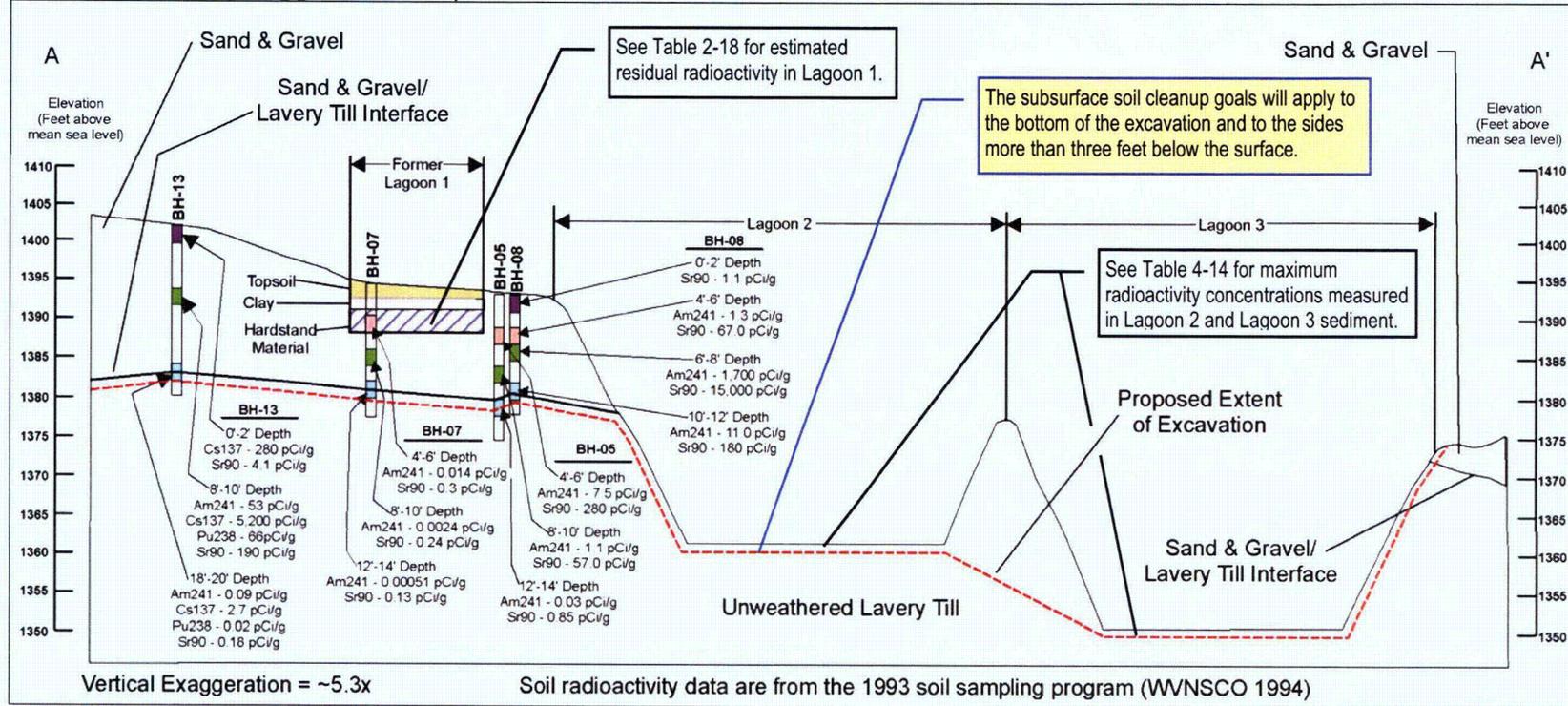


Figure 5-3. Conceptual Cross Section View of WMA 2 Excavation With Representative Data on Subsurface Soil Contamination (See Section 4.2 for more data and 7 for excavation details.)

WVDP PHASE 1 DECOMMISSIONING PLAN

Available data on radioactive contamination in subsurface soil in WMA 1 described in Section 4.2 show Sr-90 to be the dominant radionuclide at depth. Figure 4-8 shows key data, which include three samples from several feet into the unweathered Lavery till that show Sr-90 concentrations of 13 pCi/g, 41 pCi/g, and 59 pCi/g at depths in the 35 to 40 feet range.

Other radionuclides with measured above-background concentrations in subsurface soil in WMA 1, with their maximum concentrations and the associated sample depth, include: Tc-99 (19 pCi/g at 19-23 feet), Cs-137 (31 pCi/g, at 27 to 29 feet), Pu-241 (15 pCi/g at 21 to 23 feet), and Am-241 (0.1 pCi/g, 19 to 23 feet). Table 5-1 shows the maximum measured radionuclide concentrations in the Lavery till in the areas of the large excavations in WMA 1 and WMA 2. Data in the Lavery till in these areas are limited – the complete set of data is provided in Table C-4 of Appendix C.

Table 5-1. Measured Maximum Lavery Till Radionuclide Concentrations⁽¹⁾

Nuclide	WMA 1 Excavation Area		WMA 2 Excavation Area	
	Result (pCi/g)	Depth (ft)	Result (pCi/g) ⁽³⁾	Depth (ft)
C-14	1.1E-01 ⁽²⁾	38-40	none	none
Sr-90	5.9E+01 ⁽⁴⁾	38.5-39	8.5E-01	12-14
Tc-99	<5.5E-01 ⁽²⁾	37-39	none	none
I-129	<2.9E-01 ⁽²⁾	38-40	none	none
Cs-137	3.9E+00 ⁽²⁾	38-40	4.5E-01	12-14
U-232	4.1E-02	24-26	1.2E-02	12-14
U-233/234	2.3E+00 ⁽²⁾	38-40	1.8E-01	12-14
U-235	1.4E-01 ⁽³⁾⁽⁵⁾	24-26	<5.9E-03	12-14
Np-237	<2.1E-02 ⁽²⁾	37-39	none	none
U-238	1.4E+00	41-43	1.1E-01	12-14
Pu-238	<2.3E-02 ⁽²⁾	38-40	1.0E-02	12-14
Pu-239/240	<6.4E-02 ⁽²⁾	38-40	<5.9E-03	12-14
Pu-241	<5.7E-01 ⁽²⁾	38-40	<1.3E+00	12-14
Am-241	<1.3E-01 ⁽²⁾	38-40	3.0E-02	12-14
Cm-243/244	<2.3E-02 ⁽²⁾	38-40	none	none

NOTES: (1) See Table C-4 for the complete data set, which includes samples at nine locations entirely within the unweathered Lavery till within the WMA 1 excavation area. Based on boring log data, only one sample (BH-05) taken within the WMA 2 excavation area contained only unweathered Lavery till soil; the others contained some soil from the sand and gravel layer.

(2) Data are from the 2008 north plateau groundwater plume Geoprobe[®] investigation described in Section 4, with the highest non-detection values recorded (with amended sample 7608 results).

(3) Data are from sample BH-05 collected during the 1993 RCRA facility investigation described in Section 4.

(4) Data are from point GP3098 from the 1998 north plateau Geoprobe[®] sampling described in Section 4.

(5) U-235/U-236 result.

Additional Characterization Planned

The characterization program described in Section 9 will provide additional data on radioactivity in subsurface soil in WMA 1 and WMA 2 and lagoon sediment in WMA 2.

The actual depth of the WMA 1 excavation will extend at least one foot into the unweathered Lavery, and this is where the subsurface soil cleanup goals will apply, as explained in Section 7. The configuration of the residual source will therefore be similar to the bottom of the excavation shown in the representative cross section in Figure 5-2.

Figure 5-1 also shows the approximate location of the major excavation in WMA 2. As explained in Section 1 and detailed in Section 7, a single excavation will be made to remove Lagoons, 1, 2, and 3, the interceptors, the Neutralization Pit, and the Solvent Dike. The area of this excavation will be approximately 4.2 acres and its depth will vary from approximately 12 feet on the southwest end to approximately 26 feet on the northeast end.²

Figure 5-3 shows a conceptual cross section of the WMA 2 excavation. This figure also shows representative data on subsurface radioactivity. As indicated on the figure, Table 2-18 provides an estimate of residual radioactivity in Lagoon 1 and Table 4-14 shows maximum radionuclide concentrations measured in sediment in Lagoon 2 and Lagoon 3.

As indicated in order-of-magnitude estimates in Table 2-18, Cs-137 (at 510 curies) is expected to dominate the radioactivity in Lagoon 1. Other radionuclides expected to be present include Pu-241 (134 curies), Sr-90 (17 curies), and Pu-238 (6.4 curies). Table 4-14 shows significant concentrations of Sr-90, Cs-137, Pu-238, Pu-239/240, and Am-241 in Lagoon 2 sediment and lower concentrations of these radionuclides in Lagoon 3 sediment.

The actual depth of the WMA 2 excavation will extend at least one foot into the unweathered Lavery, and this is where the subsurface soil cleanup goals will apply, as explained in Section 7. In the cases of Lagoon 2 and Lagoon 3, the excavation will extend approximately two feet below the bottom the lagoons, which extend into the Lavery till. The configuration of the residual source will therefore be similar to the bottom of the excavation shown in the representative cross section in Figure 5-3.

While the subsurface soil cleanup goals serve as the remediation criteria for the two excavations as specified in Section 7, actual residual contamination levels in the Lavery till are expected to be well below these criteria. The concentrations of Sr-90 and Cs-137 are expected to be of the same order of magnitude as the lower surface soil cleanup goals. This conclusion is based on contamination data shown in Table 5-1 and the relative impermeability of the Lavery till to radionuclide migration compared to the sand and gravel layer above it.

² The 26-foot estimate is based on using the ground surface adjacent to Lagoon 3 as a reference point. The excavation is expected to extend several feet below the bottoms of Lagoons 2 and 3 to remove sediment with radioactivity concentrations above the cleanup goals.

Streambed Sediment

Streambed sediment refers only to sediment in Erdman Brook and the portion of Franks Creek running through the project premises. **Figure 5-12 in Section 5.2 below shows precisely where streambed sediment DCGLs apply.**

Surface soil DCGLs will be applied to sediment in ditches, **in tributaries to Erdman Brook and Franks Creek**, and in other parts of the project premises, with the subsurface soil DCGLs being applied to the bottom of Lagoons 2 and 3. Unique DCGLs are appropriate for Erdman Brook and Franks Creek because the areas of these streams would not support farming or grazing of livestock as would other areas of the project premises, owing to the steep stream banks.

Section 4.2 summarizes the limited available data on radioactivity in the sediment of Erdman Brook and the portion of Franks Creek on the project premises. Figure 4-6 shows sample locations, with five in Erdman Brook and four in Franks Creek. Table 4-22 shows the highest measured concentrations of Cs-137 and other radionuclides. The highest measured Cs-137 concentration was 100 pCi/g and the highest Sr-90 concentration was 10 pCi/g. **(However, Section 4.2 describes a hot spot found in Erdman Brook in 1990 with a gamma radiation level of 3000 μ R/h; a sample collected at that location showed 10,000 pCi/g Cs-137.)** The characterization program **described in Section 9** will provide additional data **on** radioactivity in the sediment of the two streams.

DCGLs **(cleanup goals)** for streambed sediment based on the unrestricted use criteria in 10 CFR 20.1402 **will support decision-making for Phase 2 of the decommissioning, and remediation of contaminated sediment in Erdman Brook and the portion of Franks Creek on the project premises is this were to be accomplished in Phase 2.**

5.1.3 Context for the Integrated Dose Assessment

Three sets of DCGLs have been developed as described in Section 5.2 to be applied to the particular areas of interest, that is:

- Surface soil DCGLs for surface soil and **for** sediment in drainage ditches on the project premises **and in tributaries to Erdman Brook and Franks Creek**, and for the sides of the WMA 1 and WMA 2 excavations from the ground surface to three feet below the surface;
- Subsurface soil DCGLs for the bottoms of the WMA 1 and WMA 2 excavations and for the excavation sides more than three feet below the ground surface; and
- Streambed sediment DCGLs for sediment in Erdman Brook and the portion of Franks Creek on the project premises **shown in Figure 5-12.**

Each set of DCGLs was developed as if the area of interest remediated to the applicable DCGLs were **to be** the only area to which a hypothetical future resident or recreationist might be exposed. However, it is more likely that a variety of receptors will be exposed to multiple sources under a range of land use scenarios. Considering each source

independently allows for flexibility in subsequent combined dose evaluations, as discussed further in Section 5.3.

Phase 1 and Phase 2 Sources

Inherent in the phased decision-making approach is the concept of Phase 1 and Phase 2 sources. Figure 5-4 identifies these different sources.

Phase 1 sources are those to be remediated during Phase 1 of the decommissioning: mainly the WMA 1 area and the large area in WMA 2 to be excavated. Surface soil in selected areas within the project premises may or may not be remediated in Phase 1³. Based on current characterization data, the main Phase 2 sources are the non-source area of the north plateau groundwater plume in WMA 2, WMA 4, and WMA 5; the Waste Tank Farm in WMA 3, and the NRC-Licensed Disposal Area (NDA) in WMA 7.

The table at the bottom of the Figure 5-4 shows the approximate amounts of total radioactivity in the different source areas based on estimates provided in Section 4. In this illustration, the remediated WMA 1 and WMA 2 excavated areas are the Phase 1 sources. The Waste Tank Farm, the non-source area of the north plateau groundwater plume, and the NDA are the Phase 2 sources, as is low-level contamination in streambed sediment. Low-level contamination in surface soil – which may or may not be remediated during Phase 1 – could be either be a Phase 1 (remediated) or Phase 2 (remediated or not) source, with the potential impact from this sources much smaller than for the others (with the exception of streambed sediment).

Figure 5-4 shows other features of the project premises at the conclusion of the Phase 1 decommissioning activities that could potentially influence future doses from residual radioactivity on the project premises:

- Groundwater flow, with the water table in the sand and gravel unit on the north plateau, with elevations expressed in feet above mean sea level, and the current pre-remediation general direction of groundwater illustrated on the figure;
- The full-scale Permeable Treatment Wall; and
- The hydraulic barrier walls to be installed during Phase 1 of the decommissioning as described in Section 7 and the French drain to be emplaced upgradient of the WMA 1 hydraulic barrier wall.

The effectiveness of these features impacts potential future doses to the receptor and overall contribution to the evaluation of combined dose from all sources.

³ As noted in Section 7.11, surface soil in selected areas of the project premises may be remediated during the Phase 1 decommissioning activities to ensure that surface soil cleanup goals are achieved in these areas.

WVDP PHASE 1 DECOMMISSIONING PLAN

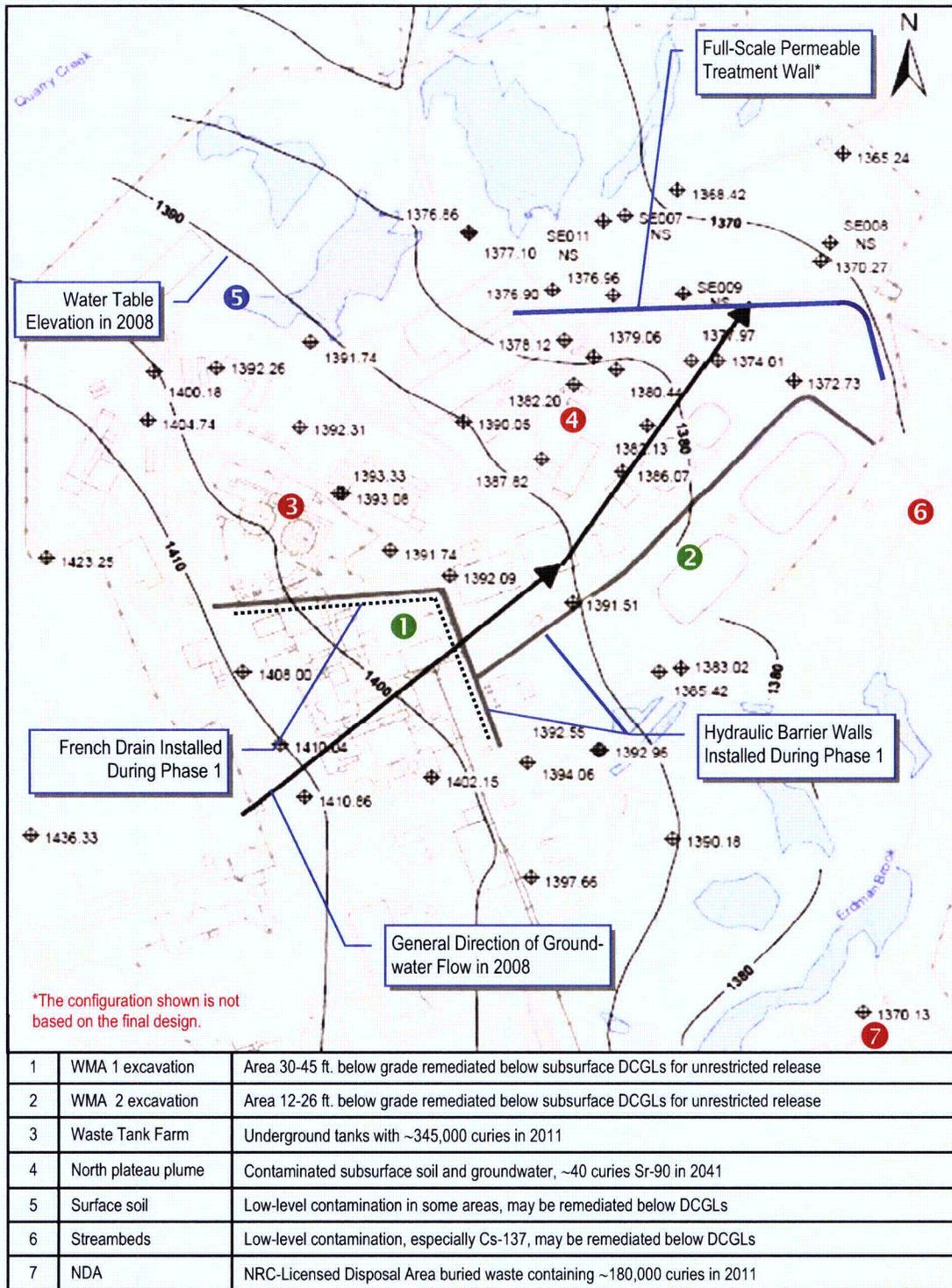


Figure 5-4. Sources at the Conclusion of Phase 1 of the Decommissioning

Potential Conditions at the Conclusion of the WVDP Decommissioning

To determine whether criteria used in Phase 1 remediation activities could potentially limit the decommissioning options for Phase 2 of the decommissioning, consideration must be given to potential approaches to Phase 2. The Decommissioning EIS evaluates a range of closure alternatives. Two of these alternatives provide bounding conditions for assessment of whether the criteria used for Phase 1 remediation activities could limit Phase 2 options:

- The site-wide close-in place-alternative, where the major facilities would be closed in place, with residual radioactivity in the Waste Tank Farm and the NDA being isolated by engineered barriers and the non-source areas of the north plateau groundwater plume being allowed to decay in place; and
- The site-wide removal alternative, where the Phase 2 sources would be removed and the entire site remediated to the unrestricted release criteria of 10 CFR 20.1402.

Compatibility of Phase 1 Remediation With the Site-Wide Close-In-Place Alternative

With the site-wide close-in place-alternative, the Phase 2 source areas would likely remain under NRC license. With Phase 1 of the decommissioning being accomplished, the contamination remaining in the WMA 1 and WMA 2 excavations will be residual radioactivity at concentrations below the subsurface soil cleanup goals located far below the surface and covered with uncontaminated earth.

Under a site-wide close-in-place approach, the remediated Phase 1 areas would be expected to fall within the controlled licensed area because of their close proximity to the Phase 2 source areas. In view of this situation, the remediation of the Phase 1 areas to unrestricted release standards would clearly be compatible with the Phase 2 source areas remaining under license. That is, remediation of the Phase 1 source areas as planned will have no impact on the site-wide close-in place-alternative and will not limit its implementation in any way.

Compatibility of Phase 1 Remediation With the Site-Wide Removal Alternative

Under the site-wide removal alternative, the Phase 2 source areas would be remediated to unrestricted release standards like the Phase 1 source areas. All of the associated radioactive waste will be disposed of offsite. However, while the remediation standards will be the same, the critical group for potential future exposures will not be the same for all parts of the site. Because remediation to unrestricted release standards under Phase 1 of the decommissioning does not preclude achievement of unrestricted release standards under Phase 2, all remedial options may be considered.

However, this situation requires consideration of potential exposures to members of the different critical groups, a matter which is addressed below.

Critical Group

Critical Group means the group of individuals reasonably expected to receive the greatest exposure to residual radioactivity for any applicable set of circumstances (10 CFR 20.1003).

Section 5.2 describes the critical groups for development of the different DCGLs. The average member of the critical group for development of the surface soil and subsurface soil DCGLs is a resident farmer. (Alternative scenario analyses described in Section 5.2 also evaluate exposure to a residential gardener.) The average member of the critical group for development of the streambed sediment DCGLs is a recreationist, that is, a person who would spend time in the Erdman Brook and Franks Creek areas engaged in activities such as fishing and hiking.

One reasonably foreseeable set of circumstances would involve a person engaged in farming at some time in the future on one part of the remediated project premises who also spends time fishing and hiking at Erdman Brook and Franks Creek. This scenario would involve an individual being exposed to two different remediated source areas and being a member of the two different critical groups. Because this scenario is not considered in development of the DCGLs for the different areas of interest, it would be appropriate to consider whether it could result in such a hypothetical individual exceeding the unrestricted dose limit, that is, 25 mrem in one year, and whether the residual radioactivity has actually been reduced to levels that are ALARA in accordance with 10 CFR 20.1402.

Considering the foregoing discussion, Section 5.3 evaluates the potential impacts of this set of circumstance (combined sources of dose to a single receptor) on the DCGLs and the associated cleanup goals to be used to guide remediation during Phase 1 of the decommissioning.

Two other factors that could potentially affect potential future doses from the remediated Phase 1 areas would be long-term erosion and potential changes in groundwater flow.

5.1.4 Potential Impact of Long-Term Erosion

The potential impact of long-term erosion is a consideration in development of DCGLs for Phase 1 of the decommissioning and for estimating potential future doses from different parts of the project premises assuming that the entire site would be remediated for unrestricted use.

Section 3.5.3 of this plan describes the site geomorphology, including erosion processes such as channel incision, slope movement, and gully formation. Table 3-13 provides information on site erosion rates from various sources.

Detailed erosion studies performed in support of the Decommissioning EIS are described in Appendix F to that document. This appendix describes past studies and recent analyses that made use of the CHILD landscape evolution model, which was calibrated for the site using a probabilistic process.

The CHILD model was used for 26 forward-in-time simulations to predict erosion rates at the WVDP over a 10,000-year time period. The models generally predicted minimal erosion on the central portion of the north plateau, gully development along the north plateau rim, and active erosion along the steep valley sides of Erdman Brook and Franks Creek. In the more erosive north plateau scenarios, gullies were predicted to advance within 328 to 656 feet of the Process Building area within the 10,000 year simulation period.

Limited field data showing actual sheet and rill erosion rates are available as indicated in Table 3-13. The maximum measured erosion among 19 measurements over an 11-year period ending in 2001 was 0.04 feet (approximately 0.5 inch) on the slope of a gully. One spot south of Lagoon 2 showed buildup of 0.04 feet (about 0.5 inch) during that period.

Conclusions that can be drawn from the available field data and the erosion studies detailed in Appendix F of the Decommissioning EIS include:

- The central portion of the north plateau is expected to be generally stable over the next 1000 years;
- The WMA 2 area, which is near the Erdman Brook stream valley, is more susceptible to erosion than the WMA 1 area;
- Existing gullies will propagate, becoming deeper and longer, and new gullies will form, mainly on the edges of the north plateau, if erosion **proceeds** unchecked;
- Rim widening and channel downcutting could occur in Erdman Brook and Franks Creek;
- With unmitigated erosion, gullies could eventually extend into the areas of Lagoons 1, 2, and 3 during the 1000-year evaluation period; and
- With unmitigated erosion, rim widening and downcutting of Erdman Brook could possibly impact the eastern edge of the areas of these lagoons, especially Lagoon 3.

These projections formed the basis for the alternate conceptual models involving erosion that are described in Section 5.2.

5.1.5 Potential Changes in Groundwater Flow Fields

Changes in the groundwater flow pattern that might result from installation of the hydraulic barriers shown in Figure 5-1 could increase the potential for recontamination of the areas remediated in Phase 1. Groundwater in the sand and gravel unit on the north plateau currently flows northeast as indicated on Figure 5-4. With this flow pattern, and with the WMA 1 and WMA 2 hydraulic barriers remaining in place, the potential for transport of contaminants by groundwater into the WMA 1 and WMA 2 areas remediated during Phase 1 of the decommissioning from Phase 2 source areas is low.

Appendix D describes the results of an analysis performed to evaluate groundwater flow conditions near these engineered barriers. This analysis suggests that the potential for recontamination of the remediated WMA 1 and WMA 2 areas will not be significantly increased with the engineered barriers in place.

5.1.6 Seepage of Groundwater

Figure 5-5 shows the locations of groundwater seeps on the north plateau. As can be seen in the figure, any groundwater from the seeps located on the project premises runs into Erdman Brook or Franks Creek (Dames and Moore 1994).

WVDP PHASE 1 DECOMMISSIONING PLAN

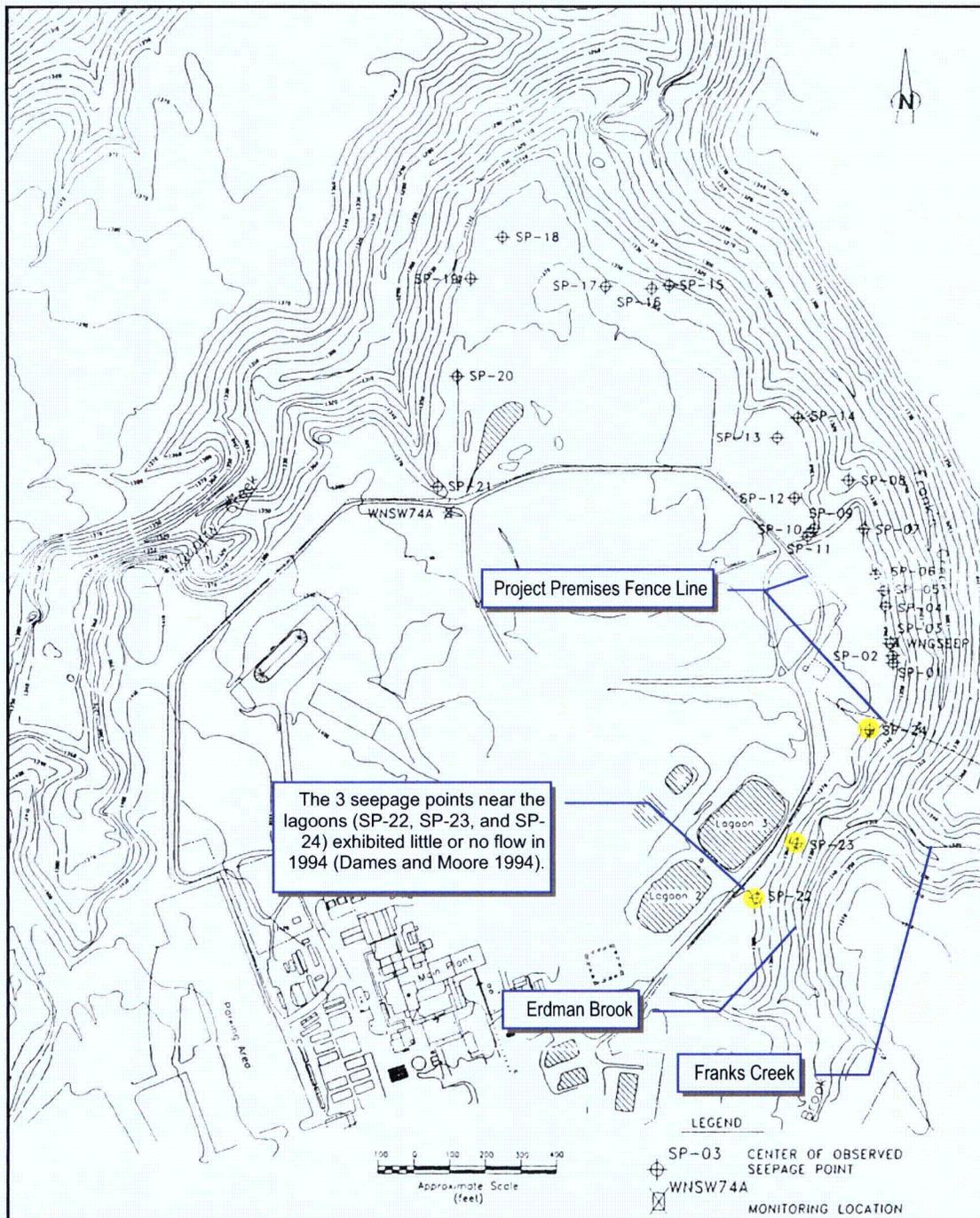


Figure 5-5. Locations of Perimeter Seeps on the North Plateau (From Dames and Moore 1994)

One other factor that could possibly affect conditions following Phase 1 of the decommissioning is seepage of radioactively contaminated groundwater into Erdman Brook and Franks Creek.

As noted **previously, streambed** sediment **will not** be remediated during Phase 1 of the decommissioning. The presence of groundwater seeps in the Erdman Brook area **was** one factor taken into account in **the decision not** to proceed with this remediation **during Phase 1**, since these seeps could possibly result in recontaminating the sediment in Erdman Brook.⁴

However, the potential for significant radioactivity in seeps in this area following Phase 1 of the decommissioning will be low due to the following factors:

- Any residual radioactivity that might remain in the Lavery till at the bottom of the remediated WMA 2 excavation will be at very low concentrations; and
- Groundwater flow changes with the Phase 1 vertical hydraulic barriers in place, as described in Appendix D, will be expected to substantially reduce the potential for contamination from the non-source area of the north plateau groundwater plume seeping into Erdman Brook.

Another factor that **was** taken into account in **the decision to not** proceed with remediation of sediment in Erdman Brook and in the portion of Franks Creek on the project premises during Phase 1 of the decommissioning **was** surface water runoff, especially runoff from the two radioactive waste disposal areas on the south plateau. Surface water runoff from both waste disposal sites is potentially contaminated due to surface soil contamination in these areas, although the potential impact on the streams is limited so long as the geomembrane covers for the waste disposal sites **remain** intact.

Note that Table D-4 in Appendix D provides flow balance estimates for post-Phase 1 conditions. These estimates do not show an increase in downward groundwater flow to the Kent Recessional Sequence following Phase 1 of the decommissioning.

5.1.7 Potential Impacts on the Kent Recessional Sequence

The potential for impacts on groundwater in the Kent Recessional Sequence from any residual radioactivity that might remain in the bottom of the WMA 1 and WMA 2 excavated areas has been evaluated and found to be very low.

Groundwater in the sand and gravel unit generally flows to the northeast across the north plateau towards Franks Creek as shown in Figure 5-4. Water balance estimates (Yager 1987 and WVNSCO 1993a) suggest that approximately 60 percent of the groundwater from the sand and gravel unit discharges to Quarry Creek, Franks Creek, and Erdman Brook through surface water drainage discharge points and the groundwater seeps located along the margins of the north plateau that are shown in Figure 5-5.

Approximately two percent of the total discharge from the sand and gravel unit travels vertically downward to the underlying unweathered Lavery till, where groundwater flows vertically downward toward the underlying Kent Recessional Sequence at an average vertical groundwater velocity of 0.20 feet per year (WVNSCO 1993a). The unweathered Lavery till is approximately 30 to 45 feet thick below the planned WMA 1 excavation and 40 to 110 feet thick below the planned WMA 2 excavation (WVNSCO 1993b).

⁴ Seeps could also release contamination into Quarry Creek. Quarry Creek lies outside of the project premises and is not within the scope of Phase 1 decommissioning activities.

It will take approximately 200 years for groundwater to migrate through the unweathered Lavery till at WMA 1 and WMA 2 assuming a Lavery till thickness of 40 feet and an average groundwater velocity of 0.20 feet per year. Mobilization and migration of the residual radionuclide inventory at the bottom of the WMA 1 and WMA 2 excavations through the Lavery till groundwater pathway will take even longer considering the sorptive properties of the Lavery till (Table 3-20).

Short-lived radionuclides (Sr-90, Cs-137, and Pu-241) will have decayed away during these time frames. The long-lived radionuclide inventory is not an issue as the residual concentrations within the Lavery till are expected to be comparable to background concentrations for surface soil. The residual radionuclide concentrations in the Lavery till in the bottom of the WMA 1 and WMA 2 excavations are expected to be lower than those reported in Table 5-1 and will therefore not significantly impact the Kent Recessional Sequence. Groundwater reaching the Kent Recessional Sequence flows laterally to the northeast at an average velocity of 0.40 feet per year and eventually discharges to Buttermilk Creek.

The potential for impacts on groundwater in Lavery till sand has also been considered.

The Lavery till sand is located 30 to 40 feet below grade within the Lavery till and is recharged by downward groundwater flow from the Lavery till. The Lavery till sand is located south of the WMA 1 excavation (Figure 3-64) and will not be impacted by the Phase 1 excavation of WMA 1.

However, the Lavery till sand underlies approximately 15,000 square feet of the southwestern most portion of WMA 2 near the Solvent Dike (Figure 3-64). The Solvent Dike was originally excavated in 1986 and will be excavated down into the Lavery till during the excavation of WMA 2. Because any residual radionuclide concentrations are expected to be less than those reported in Table 5-1, groundwater flow from the Lavery till will not significantly impact the Lavery till sand.

Note that Section 9 provides for characterization surveys around selected Process Building foundation pilings to determine whether there might be evidence of contaminant migration along some of the pilings downward towards the Kent Recessional Sequence.

5.1.8 General Dose Modeling Process

The general process for the dose modeling described in Section 5.2 and 5.3 is illustrated in Figure 5-6.

As indicated in the figure, the process involves the following major steps:

- Calculating the DCGLs using RESRAD in the deterministic mode to produce the initial base cases;
- Performing parameter sensitivity analyses and refining the conceptual models and the DCGLs as appropriate based on the results;
- Performing a probabilistic uncertainty analysis to evaluate the degree of conservatism in model input parameters, producing probabilistic peak-of-the-mean and 95th percentile DCGLs;

WVDP PHASE 1 DECOMMISSIONING PLAN

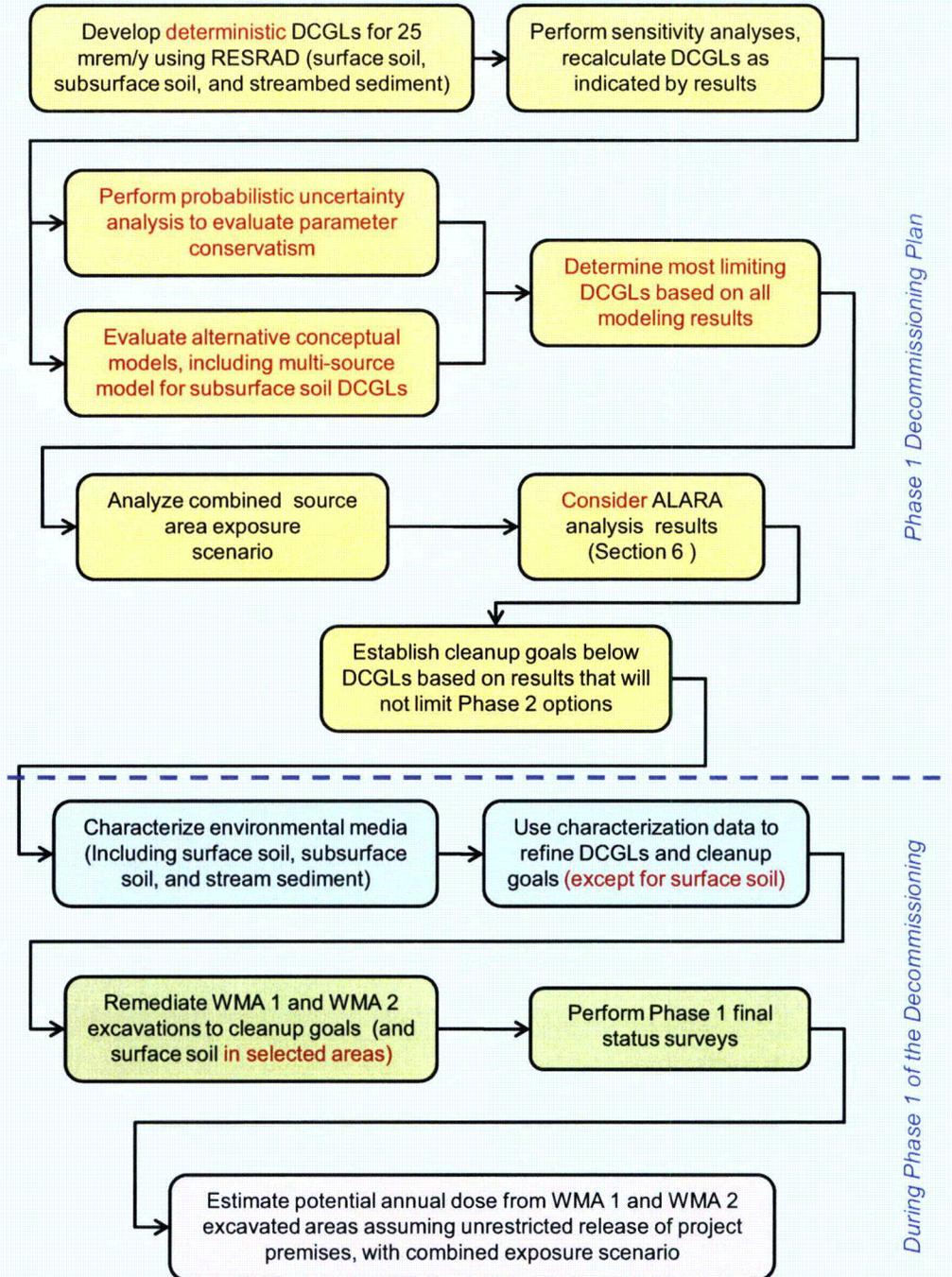


Figure 5-6. General Dose Modeling Process

- Evaluating alternate conceptual models, including a residential gardener and a multi-source conceptual model for subsurface soil DCGLs, for comparison with the initial base-case models;

- Evaluating the DCGLs produced by all of the modeling and determining the most limiting DCGLs for each radionuclide of interest; Analyzing combined source area exposure scenarios;
- Considering the results of the ALARA analysis described in Section 6;
- Establishing cleanup goals (target levels below the DCGLs) to ensure that the degree of remediation in Phase 1 of the decommissioning will not limit Phase 2 options;
- Characterizing surface soil, subsurface soil, and streambed sediment as specified in Section 9;
- Refining the DCGLs and cleanup goals based on the resulting data⁵;
- Completing remediation of the WMA 1 and WMA 2 excavations and selected surface soil areas to the cleanup goals;
- Performing Phase 1 final status surveys in the remediated Phase 1 areas, and
- Making estimates of the potential future doses for the remediated WMA 1 and WMA 2 deep excavation areas using these data.

Note that use of a surrogate radionuclide such as Cs-137 to represent all radionuclides in a mixture of radionuclides is not practical at this time because available data are not sufficient to establish radionuclide distributions in environmental media. This matter is discussed further in Section 5.4.3.

5.2 DCGL Development

This section provides the following information:

- Subsection 5.2.1 describes the conceptual models used for developing DCGLs for surface soil.
- Subsection 5.2.2 describes the conceptual models used for developing DCGLs for subsurface soil.
- Subsection 5.2.3 describes the conceptual model used for developing DCGLs for streambed sediment.
- Subsection 5.2.4 describes the mathematical model (RESRAD) used to calculate deterministic DCGLs for the various conceptual models.

⁵ The characterization to be performed as described in Section 9 will provide data on the depth and lateral extent of contamination that may be useful in better defining source geometry in the conceptual model. For example, if the actual streambed and stream bank source geometry were found to be substantially different from that assumed in the conceptual model, then the conceptual model would be revised accordingly and the DCGLs recalculated. The same approach would be used for the subsurface soil DCGLs. However, there are no plans to recalculate surface soil DCGLs for this reason because the assumed one meter source thickness is generally conservative and it is important to avoid changes to surface soil DCGLs that would impact the design of the Phase 1 final status surveys. While DCGLs are developed for 18 radionuclides, characterization data may indicate that some radionuclides may be dropped from further consideration. This could be the case, for example, if one or more of the 18 radionuclides do not show up above the minimum detectable concentration in any of the soil or sediment samples.

WVDP PHASE 1 DECOMMISSIONING PLAN

- Subsection 5.2.5 provides the modeling results – the deterministic DCGLs – along with a discussion of these results.
- Subsection 5.2.6 describes sensitivity analyses performed.
- Subsection 5.2.7 describes the probabilistic uncertainty analysis.
- Subsection 5.2.8 describes the multi-source analysis for subsurface soil DCGLs that takes into account releases of radioactivity from the bottoms of the deep excavations by diffusion.

The DCGL development analyses simulate the behavior of residual radioactivity over 1000 years, a period during which peak annual doses from the radionuclides of primary interest would be expected to occur. DCGLs have been developed for residual radioactivity that will result in 25 mrem per year dose to the average member of the critical group for each of the following 18 radionuclides of interest:

Am-241	Cs-137	Pu-239	Tc-99	U-235
C-14	I-129	Pu-240	U-232	U-238
Cm-243	Np-237	Pu-241	U-233	
Cm-244	Pu-238	Sr-90	U-234	

Early studies related to the long-term performance assessment for residual radioactivity at the site included consideration of the initial inventory of radionuclides received on site and their progeny. This list was screened to eliminate short-lived radionuclides and those radionuclides present in insignificant quantities. Thirty radionuclides of interest remained after this screening process. These radionuclides were important to worker dose and/or long-term dose from residual radioactivity.

In characterization of radionuclides in the area of the Process Building, the north plateau groundwater plume, and the lagoons, it was determined that 18 of the 30 radionuclides were important for the development of Phase 1 DCGLs. These radionuclides were selected based on screening of simplified groundwater release and intrusion scenarios for north and south plateau facilities. The screening indicated that other radionuclides will in combination contribute less than one per cent of potential dose impacts at the individual facility.

The list of radionuclides for which DCGLs are initially developed will be expanded if necessary following completion of soil and sediment characterization described in Section 9. If other radionuclides show up in concentrations significantly above the minimum detectable concentrations, additional DCGLs will be developed for these radionuclides and their progeny, as appropriate. Conversely, if any of the 18 radionuclides of interest fail to show up in concentrations above the minimum detectable concentrations, then they may be omitted from the final DCGLs for the Phase 1 actions.

As explained in Section 1, the DCGLs for Sr-90 and Cs-137 were developed to incorporate a 30-year decay period from 2011. That is, achieving residual radioactivity levels less than the DCGLs will ensure that dose criteria of 10 CFR 20.1402 will be met in

2041.⁶ Although a 30-year decay period could have been applied to all radionuclides, Sr-90 and Cs-137 were selected based on their prevalence in soil and sediment contamination, their expected peak doses at the onset of exposure, and the short half lives of these particular radionuclides.

5.2.1 Conceptual Models for Surface Soil DCGL Development

The initial base-case conceptual model for development of surface soil DCGLs is described first.

Surface Soil Conceptual Model (Base-Case)

Figure 5-7 illustrates the conceptual model for surface soil DCGL development. As is evident from this figure, which was adapted from the RESRAD Manual (Yu, et al. 2001), the basic RESRAD model is used.

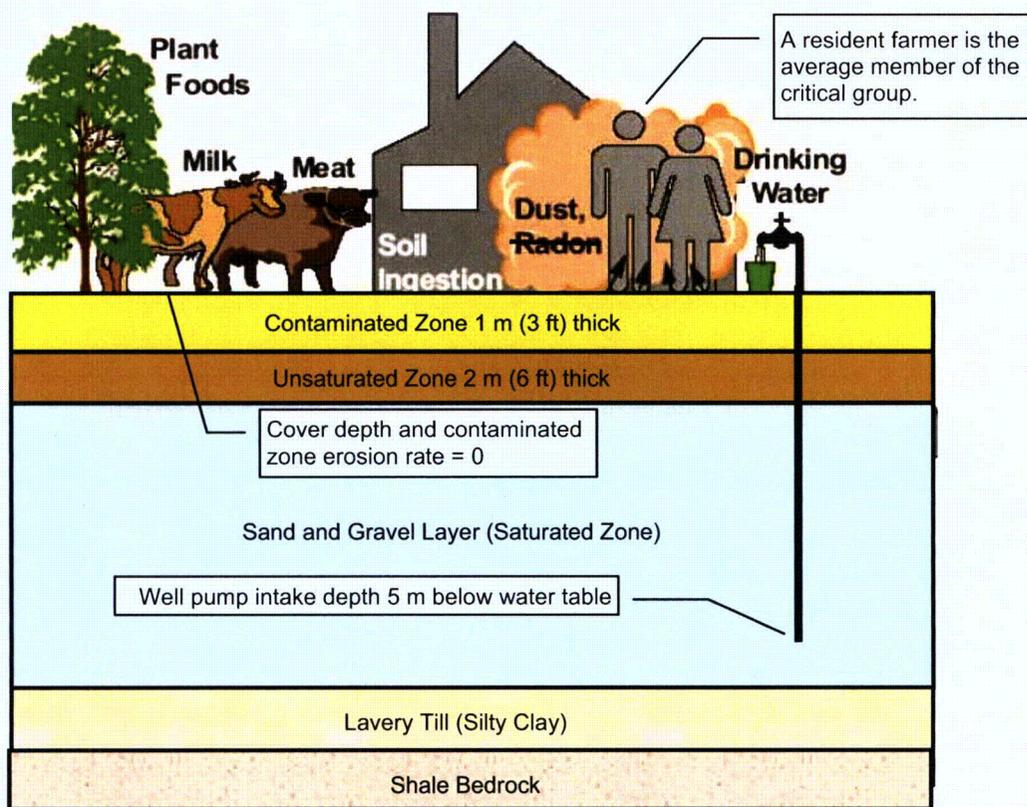


Figure 5-7. Conceptual Model for Surface Soil DCGL Development

⁶ This approach will support any license termination actions that may take place in Phase 2 of the decommissioning. As noted previously, the decision on the Phase 2 decommissioning approach could be made within 10 years of the Record of Decision and Findings Statement documenting the Phase 1 decisions. If this approach were to involve unrestricted release of the site, achieving this condition would be expected to take at least another 20 years due to the large scope of effort to exhume the underground waste tanks and the NDA. It is therefore highly unlikely that conditions for unrestricted release of the project premises could be established before 2041. If Phase 2 were to involve closing radioactive facilities in place, then institutional controls would remain in place.

WVDP PHASE 1 DECOMMISSIONING PLAN

RESRAD is a computer model designed to estimate radiation doses and risks from RESidual RADioactive materials (Yu, et al. 2001). DOE Order 5400.5 designates RESRAD for the evaluation of radioactively contaminated sites, and NRC has approved the use of RESRAD for dose evaluation by licensees involved in decommissioning. RESRAD capabilities are discussed further in Section 5.2.2.

A resident farmer is the average member of the critical group for development of surface soil DCGLs. The hypothetical residence and farm are assumed to be located on a part of the project premises impacted solely by radioactivity in surface soil.

Other possible critical groups were considered. However, a resident farmer was **assumed** to be most limiting because such an individual would be engaged in a wider range of activities that could result in greater exposure to residual radioactivity in surface soil than other critical groups considered. **(This assumption was confirmed by evaluation of alternate conceptual models involving erosion and a residential gardener as discussed below.)**

The resident farmer would be impacted by a number of exposure pathways with long exposure durations. This hypothetical individual would utilize significant amounts of groundwater that involves consideration of secondary exposure pathways such as household water use, irrigation, and watering livestock. The resident farmer scenario also is consistent with current and projected future land uses for Cattaraugus County as discussed in Section 3.

Note that the geological units shown in Figure 5-7 are representative models of the north plateau as shown in Figure 3-6. Figure 3-7 shows that the geological units on the south plateau are different in that the sand and gravel unit does not extend to that area. However, DCGLs developed using the conceptual model illustrated in Figure 5-7 are appropriate for surface soil on the south plateau because the input parameters used in the modeling for the north plateau will generally be conservative for the south plateau. For example, site-specific distribution coefficients for the sand and gravel unit (where available) are typically lower than those for the Lavery till, and use of the lower values results in **less resistance to** radionuclide movement through soil, **allowing** less time for radioactive decay to take place.⁷

Table 5-2 shows the exposure pathways evaluated for development of the surface soil DCGLs.

⁷ Table C-2 of Appendix C shows that site-specific K_d values for neptunium, plutonium, and strontium in the sand and gravel unit are used in the surface soil model. Table 3-20 shows the basis for these values. **The use of lower K_d values than those in south plateaus soil is conservative for water pathways, but may not be conservative for plant uptake and direct exposure pathways. However, the model would be conservative for south plateau conditions for most radionuclides.**

Table 5-2. Exposure Pathways for Surface Soil DCGL Development

Exposure Pathways	Active
External gamma radiation from contaminated soil	Yes
Inhalation (airborne radioactivity from re-suspended contaminated soil)	Yes
Plant ingestion (produce impacted by contaminated soil and groundwater sources)	Yes
Meat ingestion (beef impacted by contaminated soil and groundwater sources)	Yes
Milk ingestion (impacted by contaminated soil and groundwater sources)	Yes
Aquatic food ingestion	No ⁽¹⁾
Ingestion of drinking water (groundwater impacted by contaminated soil)	Yes
Ingestion of drinking water (from surface water) ⁽²⁾	No
Soil ingestion (while farming and residing on contaminated soil)	Yes
Radon inhalation	No ⁽³⁾

NOTES: (1) Fish ingestion is considered in development of the streambed sediment DCGLs and in the combined scenario discussed in Section 5.3.

(2) Groundwater was assumed to be the source of all drinking water because the low flow volumes in Erdman Brook and Franks Creek could not support the resident farmer. Also, use of surface water would not be as conservative as groundwater since surface water is diluted by runoff from the entire watershed area. Incidental ingestion of water from the streams is evaluated in development of the streambed sediment DCGLs as shown in Table 5-6.

(3) For the standard resident farmer scenario, the radon pathway is not considered (Appendix J, NRC 2006).

RESRAD requires a variety of input parameter values to completely describe the conceptual model. All of the input parameters for development of the surface soil DCGLs appear in Appendix C. Table 5-3 identifies selected key input parameters.

Table 5-3. Key Input Parameters for Surface Soil DCGL Development⁽¹⁾

Parameter (Units)	Value	Basis
Area of contaminated zone (m ²)	1.0E+04	Necessary for subsistence farming.
Thickness of contaminated zone (m)	1.0E+00	Conservative assumption. ⁽²⁾
Cover depth (m)	0	Contamination on surface.
Contaminated zone erosion rate (m/y)	0	Conservative assumption. ⁽³⁾
Well pump intake depth below water table (m)	5.0E+00	Consistent with water table.
Well pumping rate (m ³ /y)	5.72E+03	See Table C-2.
Unsaturated zone thickness (m)	2.0E+00	Typical for north plateau.
Distribution coefficient for strontium (mL/g)	5.0E+00	See Table C-2.
Distribution coefficient for cesium (mL/g)	2.8E+02	See Table C-2.
Distribution coefficient for americium (mL/g)	1.9E+03	See Table C-2.

NOTES: (1) See Appendix C for other input parameters. Metric units are used here because they are normally used in RESRAD.

WVDP PHASE 1 DECOMMISSIONING PLAN

- (2) Available data discussed in Sections 2.3.2 and 4.2 suggest that most contamination will be found within a few inches of the surface except where the north plateau groundwater plume has impacted subsurface soil. The one meter thickness is an appropriate compromise for the set of radionuclides of interest whose primary dose pathways range from direct exposure, to groundwater ingestion, to plant uptake.
- (3) This assumption is conservative because it results in no depletion of the source through erosion.⁸

Key features of this conceptual model and key assumptions include:

- The areal extent of surface soil contamination, which has not been well defined, can be represented by a distributed source spread over a relatively large area (10,000 square meters or approximately 2.5 acres);
- The average depth of contamination (contamination zone thickness) is approximately 3.3 feet (one meter), a conservative assumption for the site;
- Because the model considers only surface contamination, the resulting DCGLs and cleanup goals are applicable only to portions of the project premises where there is no subsurface contamination (i.e., contamination does not extend beyond a depth of 1 meter);
- All water use (e.g., household, crop irrigation, and livestock watering) is from contaminated groundwater;
- Adequate productivity from a well pumping from the aquifer will be available in the future to support a subsistence farm;
- Soil erosion (i.e., source depletion) does not occur over the 1,000-year modeling period;
- The non-dispersion groundwater model is used because of the large contaminated area consistent with applicable guidance (Yu, et al. 2001, Appendix E);
- The groundwater flow regime under the post-remedial conditions is unchanged from the current configuration (e.g. flow direction, aquifer productivity); and
- DCGLs that reflect 30 years of decay (i.e., apply to the year 2041) are appropriate for Sr-90 and Cs-137. Although a 30-year decay period could have been applied to all radionuclides, Sr-90 and Cs-137 were selected based on their prevalence in surface soil, their expected peak doses at the onset of exposure, and the short half lives of these particular radionuclides, as noted previously.

Alternate Conceptual Model for Surface Soil DCGLs (Erosion, Offsite Receptor)

Other conceptual models were considered, even though the resident farmer model with its many exposure pathways is generally considered to be the most conservative model. To

⁸ The conservative nature of the assumption can be demonstrated by assuming that erosion takes place and evaluating potential doses to a receptor located in a gully where radioactivity has been displaced by erosion. As explained in the discussion of alternate conceptual models below, the receptor in the area of the gully would receive less dose on an annual basis than would the resident farmer due to factors such as source dilution, spending less time in the contaminated area, and receiving exposure through fewer pathways. Consideration of potential doses to an offsite receptor from radioactivity displaced to the stream through erosion indicates that there is a reasonable expectation that offsite doses would not be significant either.

confirm that the assumption of no erosion in the contamination zone (one of the key parameters in Table 5-3) is conservative, an analysis was performed to estimate the potential doses to an offsite receptor from radioactivity that could be released from the hypothetical garden used in the base-case model through erosion.

In this analysis, eroded soil was assumed to be transported in surface water to a receptor located on Cattaraugus Creek near the confluence with Buttermilk Creek who ingested both the water and fish harvested from the water and used the water to irrigate a garden. The results showed that doses to this receptor would be insignificant.

Alternate Conceptual Model for Surface Soil DCGLs (Residential Gardener)

Another alternative exposure scenario was evaluated to confirm that the base-case resident farmer scenario is bounding for development of surface soil DCGLs. This alternative scenario involved a residential gardener scenario.

The receptor in the residential gardener scenario is a hypothetical person who resides in the area and grows a vegetable garden. This scenario differs from the resident farmer scenario in that the person of interest does not consume meat or milk produced on the property and spends less time outdoors in the hypothetical garden. The well pumping rate used in this scenario was lower than that used in the resident farmer model (1140 cubic meters per year compared to 5720 meters per year) to reflect the smaller garden being used and the lower well water usage.

This alternative exposure scenario produced DCGLs that were slightly higher than those produced by the base-case resident farmer model for all 18 radionuclides. Consequently, the base-case model is bounding for surface soil DCGL development when compared to the residential gardener scenario. (See Section 5.2.7 for the results of the probabilistic uncertainty analysis.)

5.2.2 Subsurface Soil Conceptual Models

Evaluation of Various Subsurface Soil Conceptual Models

The analyses described in Revision 0 and Revision 1 to this plan made use of the base-case conceptual model for subsurface soil DCGL development described below and illustrated in Figure 5-8. Minor changes were made to this conceptual model in Revision 2 that produced DCGLs that were slightly higher for most radionuclides.

Additional analyses were also performed to determine whether this conceptual model, which makes use of the resident farmer scenario, represented the bounding case for potential future doses from the remediated deep excavations. These additional analyses, which are described below, involved:

- Evaluating the potential acute dose to the hypothetical individual drilling the well (the two meter diameter cistern) used in the original base case model,
- Evaluating potential acute dose to a hypothetical individual who might drill a natural gas well in the area of one of the deep excavations,
- Evaluating potential doses to a recreational hiker in the area of the lagoons in WMA 2 assuming that unchecked erosion would eventually produce deep gullies in this area,
- Evaluating potential doses to an offsite receptor from residual radioactivity at the bottom of the deep excavation in WMA 2 that might be released to Erdman Brook if deep gullies were to eventually cut into this area, and
- Evaluating a residential gardener scenario.

Of these five alternate conceptual models, one, the residential gardener model, was found to be more limiting for some radionuclides than the original base-case resident farmer scenario.

To help determine whether the input parameters used in the original base-case model were sufficiently conservative, a comprehensive probabilistic uncertainty analysis was performed (similar analyses were also performed for surface soil and streambed sediment DCGL development). Section 5.2.7 describes this analysis. The resulting peak-of-the-mean DCGLs were somewhat lower for most radionuclides than the DCGLs produced by the deterministic resident farmer and residential gardener scenarios.

Another analysis was performed to evaluate whether continuing release of residual radioactivity from the bottom of the deep excavations would influence potential future doses from the remediated deep excavations. Section 5.2.8 describes this analysis. The original base-case conceptual model was modified to add a secondary source of radioactivity from residual contamination at the bottom of the deep excavation that moves upward by diffusion and is drawn into the hypothetical well, resulting in additional dose to the resident primarily from the drinking water pathway.

This multi-source model was analyzed using the resident farmer scenario and also the residential gardener scenario, the latter with three different upper contamination zone geometries to evaluate the sensitivity of the model to the contamination zone area and thickness. The results showed that this model was more limiting for nine of the 18 radionuclides of interest than the other subsurface soil DCGL conceptual models that were evaluated.

Consideration of the results of all of this subsurface soil dose modeling led to the decision to use the lowest DCGLs among all of the modeling results as the basis for the subsurface soil cleanup goals in the interest of conservatism.

Initial Base-Case Conceptual Model

Figure 5-8 illustrates the **initial base-case** conceptual model for subsurface soil DCGL development. The basic RESRAD model is used as with development of surface soil DCGLs, with a resident farmer being the average member of the critical group. The hypothetical residence and farm are assumed to be located in the remediated WMA 1 area. Exposure to the subsurface radioactivity occurs following intrusion and surface dispersal when installing a water collection cistern.

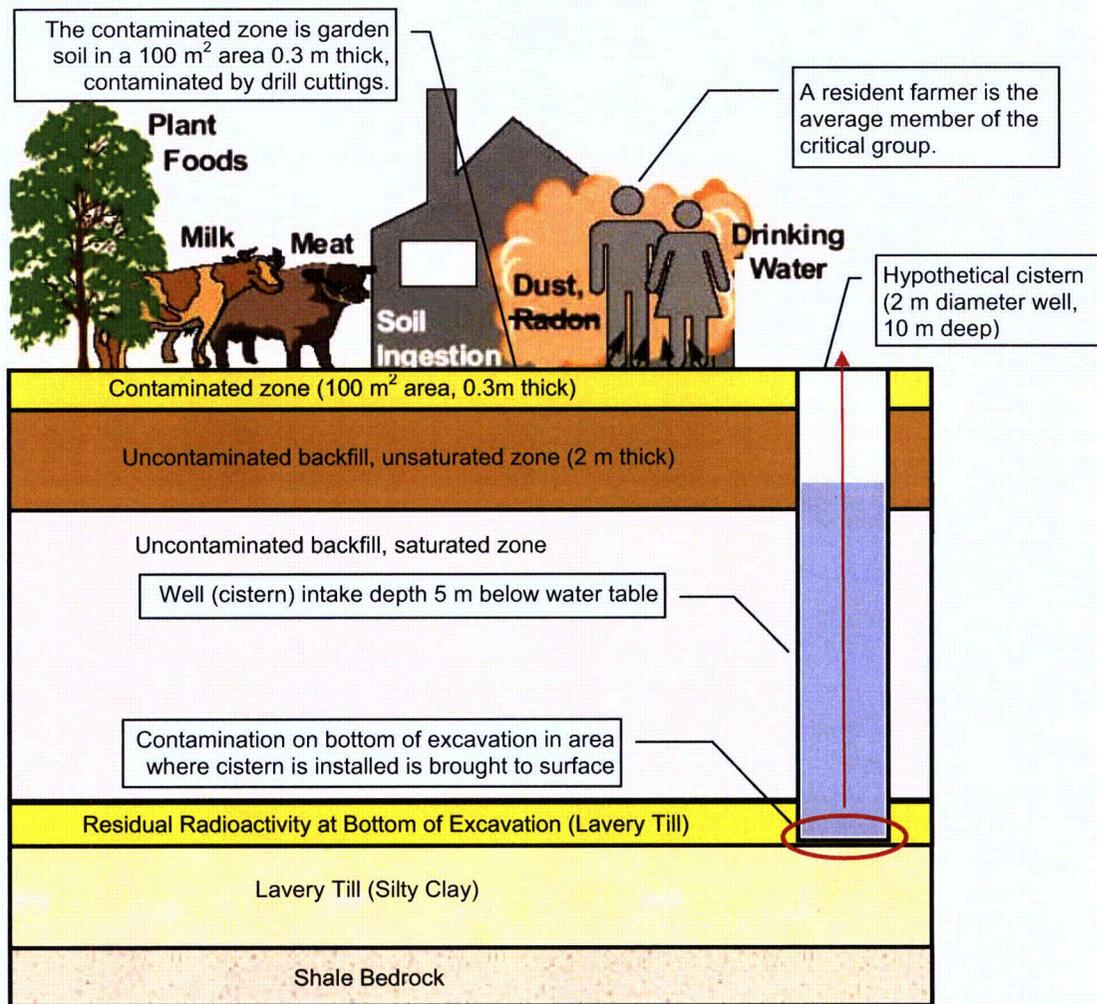


Figure 5-8. Conceptual Model for Subsurface Soil DCGL Development

Other possible critical groups were considered as with the conceptual model for surface soil DCGLs. However, a resident farmer was **initially assumed** to be most limiting because such an individual would be engaged in a wider range of activities that could result in greater exposure to residual radioactivity in subsurface soil than other critical groups considered.

Consideration was given to a home construction scenario with the basement in the hypothetical home extending 10 feet below the surface. However, this scenario was not considered to be plausible because any contaminated subsurface soil will be more than 10 feet below the surface in the remediated WMA 1 and WMA 2 areas (the bottoms of the excavations will be more than 10 feet below the surface and uncontaminated soil will be used to backfill the excavations).

Note that Section 7 specifies that the uncontaminated backfill as shown in the figure will be soil obtained from outside of the Center from an area that has not been impacted by site radioactivity. No soil removed during the excavation work will be used in filling the excavation, even if that soil were determined to be uncontaminated.

Consideration of NRC Guidance Related to Buried Radioactivity

Also considered in development of this conceptual model was NRC guidance related to assessment of buried radioactivity in Appendix J to NUREG-1757, Volume 2 (NRC 2006). This guidance applies to cases where radioactive material is buried deep enough that an external dose is not possible in its existing configuration; any radioactivity remaining at the bottom of the WMA 1 and WMA 2 excavations would meet this condition, and the WVDP situation is consistent with the intent of the guidance.

The NRC notes that a conservative analysis could be performed that assumes all of the material is spread on the surface. It describes two alternative exposure scenarios: (1) leaching of the radionuclides to groundwater, which is then used by a residential farmer, and (2) inadvertent intrusion into the buried radioactive material, with part of the radioactivity being spread across the surface where this fraction causes exposure to a resident farmer through various pathways. NRC further notes that

“The second alternative exposure scenario encompasses all the exposure pathways and, although not all of the source term is in the original position, leaching will occur both from the remaining buried residual radioactivity (if there is any) and the surface soil. Unless differences in the thickness of the unsaturated zone will make a tremendous difference in travel time to the aquifer, the groundwater concentrations should be similar and, therefore, will generally result in higher doses than the first alternate scenario.”

The surface soil DCGLs discussed previously represent the case where all of the radioactive material of interest is located on the surface; as explained in Section 6, possible application of these DCGLs to the subsurface soil of interest would be addressed in the ALARA analysis. DOE has selected the second alternative exposure scenario – inadvertent intrusion into the buried material, that is, into any residual radioactivity at the bottom of the WMA 1 and WMA 2 excavations – as the basis for development of the subsurface soil DCGLs. NRC discusses in Appendix J to NUREG-1757 (NRC 2006) the use of RESRAD in analysis of the inadvertent intrusion scenario, which DOE has implemented here.

Note that a combination of inadvertent intrusion and continuing releases from the bottoms of the remediated deep excavations was also evaluated in the multi-source conceptual model as described in Section 5.2.8,

WVDP PHASE 1 DECOMMISSIONING PLAN

This conceptual model has the following features, some of which are indicated on Figure 5-8:

- The initial modeled source of contamination brought to the surface consists of residual radioactivity in an area two meters (about six feet) in diameter and one meter (about three feet) thick, the top surface of which lies nine meters (about 30 feet) below the ground surface. The contamination assumed to be in this volume of subsurface soil represents the residual radioactivity of interest at the bottom of the WMA 1 or WMA 2 excavation. The exposure occurs when the subsurface radioactivity is deposited on the ground surface where it can result in exposure to members of the critical group through various pathways.
- For conservatism the hypothetical well is assumed to have a large diameter representative of a cistern, rather than the smaller diameter of a typical water supply well (eight inches). The larger diameter provides for a greater volume of contamination being brought to the surface, and is therefore conservative compared to the typical well diameter.
- The nine meters (about 30 feet) of uncontaminated backfill above the initial source of contamination commingles with the contaminated soil, and the mixture is assumed to uniformly cover a cultivated garden area of 100 square meters (about 1000 square feet), i.e., a small portion of the 10,000 square meter garden, to a depth of 0.3 meter (one foot).⁹
- The remainder of the contamination in the bottom of the excavation was not modeled as a continuing source to groundwater because this source is located below the assumed well pump intake depth and **was** not expected to leach upward into the source of water available to the resident farmer. **(However, additional analysis showed that doses from continuing releases from the contamination at the bottom of the excavation would be significant for some radionuclides as described in Section 5.2.8.)**

Table 5-4 shows the exposure pathways for development of the subsurface soil DCGLs, which are the same as for the surface soil DCGLs.

Table 5-4. Exposure Pathways for Subsurface Soil DCGL Development

Exposure Pathways	Active
External gamma radiation from contaminated soil	Yes
Inhalation of airborne radioactivity from re-suspended contaminated soil	Yes
Plant ingestion (produce impacted by contaminated soil and groundwater contaminated by impacted soil)	Yes
Meat ingestion (beef impacted by contaminated soil and groundwater contaminated by impacted soil)	Yes
Milk ingestion (impacted by contaminated soil and groundwater contaminated by impacted soil)	Yes
Aquatic food ingestion	No ⁽¹⁾

⁹ Note that larger contamination zone areas were evaluated in the multi-source conceptual model described in Section 5.2.8

Table 5-4. Exposure Pathways for Subsurface Soil DCGL Development

Exposure Pathways	Active
Ingestion of drinking water (from groundwater contaminated by impacted soil)	Yes
Ingestion of drinking water (from surface water) ⁽²⁾	No
Soil ingestion	Yes
Radon inhalation	No ⁽³⁾

- NOTES: (1) Fish ingestion is considered in development of the streambed sediment DCGLs and in the combined scenario discussed in Section 5.3.
- (2) Groundwater was assumed to be the source of all drinking water because the low flow volumes in Erdman Brook and Franks Creek could not support the resident farmer. Use of surface water would also not be as conservative as groundwater since surface water is diluted by runoff from the entire watershed area. Incidental ingestion of water from the streams is evaluated in development of the streambed sediment DCGLs as shown in Table 5-6.
- (3) In using the standard resident farmer scenario in modeling of buried radioactivity, the radon pathway is not considered (Appendix J, NRC 2006).

All of the input parameters for development of the subsurface soil DCGLs appear in Appendix C. Table 5-5 identifies selected key input parameters.

Table 5-5. Key Input Parameters for Subsurface Soil DCGL Development⁽¹⁾

Parameter (Units)	Value	Basis
Initial source - cistern diameter (m)	2.0E+00	Conservative values used to estimate radioactivity brought to the surface to be mixed in garden soil.
Initial source – depth below surface (m)	9.0E+00	
Initial source – thickness (m)	1.0E+00	
Area of contaminated zone (m ²)	1.0E+02	Area drill cuttings from cistern installation spread on surface.
Thickness of contaminated zone (m)	3.0E-01	Contaminated soil depth in garden.
Cover depth (m)	0	Contamination on surface.
Contaminated zone erosion rate (m/y)	0	Conservative assumption. ⁽²⁾
Well pumping rate (m ³ /y)	5.72E+03	See Table C-2.
Unsaturated zone thickness (m)	2.0E+00	Reasonable for WMA 1 and WMA 2.
Distribution coefficient for strontium (mL/g)	1.5E+01	See Table C-2.
Distribution coefficient for cesium (mL/g)	4.8E+02	See Table C-2.
Distribution coefficient for americium (mL/g)	4.0E+03	See Table C-2.

- NOTES: (1) See Appendix C for other input parameters. Metric units are used here because they are normally used in RESRAD.
- (2) This assumption is conservative because it results in no depletion of the source.¹⁰

¹⁰ The conservative nature of the assumption can be demonstrated by assuming that erosion takes place and evaluating potential doses to a receptor located in a gully where radioactivity has been exposed by erosion. As explained in the discussion of alternate conceptual models below, the receptor in the area of the gully would receive less dose on an annual basis than would the resident farmer due to factors such as spending less time in the contaminated area and receiving exposure through fewer pathways. Consideration of potential doses to an offsite receptor from radioactivity displaced to the stream through erosion indicates that there is a reasonable expectation that offsite doses would not be significant either, as discussed below.

Key assumptions associated with this conceptual model include:

- Contamination in the bottom one meter of the 10 meter deep excavation of the two meter diameter cistern would be brought to the surface, along with the overlying uncontaminated backfill, and blended into the soil over a 100 square meter area used by the resident farmer.
- All water used by the resident farmer (e.g., household, crop irrigation, and livestock watering) is groundwater which has been impacted by leaching of contaminants from surface soil (distributed excavated material) via infiltration of precipitation and irrigation water;
- Surface soil erosion (i.e., source depletion) does not occur over the 1,000 year-modeling period;
- The groundwater flow regime under the post-remedial conditions is unchanged from the current configuration (e.g. flow direction, aquifer productivity); and
- DCGLs that reflect 30 years of decay (i.e., apply to the year 2041) are appropriate for Sr-90 and Cs-137. Although a 30-year decay period could have been applied to all radionuclides, Sr-90 and Cs-137 were selected based on expected peak doses at the onset of exposure and the short half lives of these particular radionuclides, **as noted previously.**

Alternate Conceptual Model for Subsurface Soil DCGLs (Cistern Well Driller)

A drilling worker scenario evaluates dose to a hypothetical individual installing the cistern, such as from contamination brought to the surface in the form of drill cuttings that could be set aside near the cistern. A well driller scenario was evaluated **using RESRAD with conservative assumptions. Key elements in the model included:**

- **The drilling worker being exposed to excavated Lavery till material from the bottom of the excavation that was deposited on top of uncontaminated soil in the vicinity of the cistern for a 40 hour period, even though the actual exposure period would likely be much shorter;**
- **The contamination zone being nine square meters in area and 0.333 meters thick, based on an excavated volume of three cubic meters of contaminated Lavery till material; and**
- **An assumption of no water shielding, even though water in a cuttings pond would typically provide shielding from direct radiation.**

The exposure pathways considered included inadvertent ingestion of contaminated soil, inhalation of contaminated dust, and direct exposure to contaminated soil brought to the surface during the drilling. **The resulting DCGLs, which are shown in Table 5-11c in Section 5.2.8, were greater than the subsurface soil DCGLs for all radionuclides developed for the resident farmer scenario, indicating the well driller scenario is less limiting than the resident farmer scenario used in developing the subsurface soil DCGLs.**

Alternate Conceptual Model for Subsurface Soil DCGLs (Erosion, Onsite Receptor)

An alternate conceptual model was evaluated involving the potential impact of unchecked erosion in WMA 2 to an onsite receptor. The model assumed that gully erosion would produce narrow, deep steep-sided gullies, conditions where building a home and growing crops would not be practical. A plausible scenario for these conditions would involve a recreationist spending time hiking in the area, which is assumed to be rent by deep gullies that extend to the bottom of the WMA 2 excavation. Figure 5-9 illustrates the basic conceptual model. This scenario was analyzed using RESRAD in the deterministic mode.

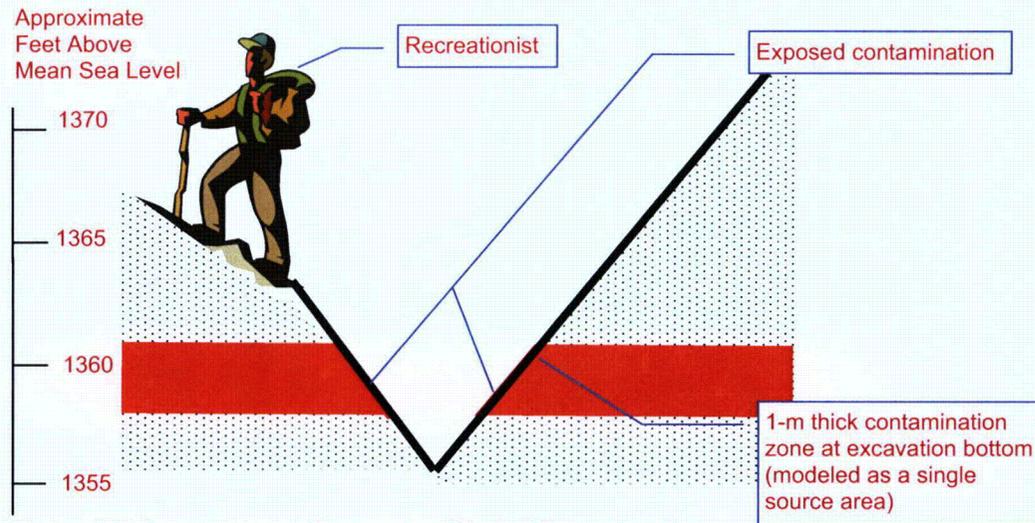


Figure 5-9 Recreationist Conceptual Model Cross Section

The modeling of this recreationist scenario produced DCGLs for 25 mrem per year that were more than one order of magnitude greater than the DCGLs produced with the initial base-case resident farmer/cistern scenario for all 18 radionuclides of interest as shown in Table 5-11c in Section 5.2.8. These results demonstrate that the resident farmer/cistern scenario is more limiting for an onsite receptor.

Alternate Conceptual Model for Subsurface Soil DCGLs (Erosion, Offsite Receptor)

Another alternative scenario was evaluated to determine the potential impact of long-term erosion in WMA 2 to an offsite receptor. This analysis estimated the potential doses to an offsite receptor from radioactivity that could be released from the bottom of the remediated WMA 2 excavation due to formation of a gully that eventually cut through the bottom of the backfilled excavation.

In this analysis, radioactivity in eroded soil from the bottom of the WMA 2 backfilled excavation was assumed to be transported in surface water to a receptor located on Cattaraugus Creek near the confluence with Buttermilk Creek who ingested both the water and fish harvested from the water and used the water to irrigate a garden. Both the area of Lagoon 1 and the area of Lagoon 3 were considered using conservative erosion rates. The results showed that doses to this receptor would be insignificant compared to the onsite receptor doses estimated in the base-case resident farmer model. Table 5-11c below shows the DCGLs calculated for the Lagoon 3 area.

Alternate Conceptual Model for Subsurface Soil DCGLs (Natural Gas Well Driller)

Installation of a natural gas well was also evaluated. Installation of this type of well would take longer than installation of a cistern because the well would be much deeper, would require well/formation development by hydrofracturing, and would require the installation of conveyance piping and valving. The analysis focused on exposure to the drilling worker. Key elements in the model included:

- The natural gas well being 0.5 meter (20 inches) in diameter and 100 meters (330 feet) deep (a conservative estimate given typical depths in excess of 1,000 meters); and
- The drilling worker being exposed to excavated Lavery till material from the bottom of the excavation that was deposited in a cuttings pit near the worker's location for 500 hours.

The exposure pathways considered included inadvertent ingestion of contaminated soil, inhalation of contaminated dust, and direct exposure to contaminated soil brought to the surface during the drilling. RESRAD version 6.4 in the deterministic mode was used to perform the calculations. The resulting DCGLs shown in Table 5-11c below were one or more orders of magnitude greater than the deterministic base-case resident farmer subsurface soil DCGLs for all radionuclides, demonstrating that the base-case resident farmer-cistern installation scenario is more limiting.

Alternate Conceptual Model for Subsurface Soil DCGLs (Residential Gardener)

Another alternative exposure scenario was evaluated to determine whether the base-case resident farmer-cistern installation scenario was bounding for development of subsurface soil DCGLs. This alternative scenario involved a residential gardener scenario.

The receptor in the residential gardener scenario is a hypothetical person who resides in the area and grows a vegetable garden. This scenario differs from the resident farmer scenario in that the person of interest does not consume meat or milk produced on the property and spends less time outdoors in the hypothetical garden. The well pumping rate used in this scenario was lower than the rate used in the resident farmer model (1140 cubic meters per year compared to 5720 meters per year) to reflect the smaller area being used and the lower well water usage.

This analysis was performed using three models which differed with respect to the area of the contamination zone and its thickness:

- Model 1 used a 100 square meter area and 0.3 meter depth, the base-case values in the base-case resident farmer deterministic analysis;
- Model 2 used a 300 square meter area and 0.1 meter depth; and
- Model 3 used a 50 square meter area and 0.6 meter depth;

This alternative exposure scenario produced DCGLs for some radionuclides that were lower than those produced by the base-case resident farmer model. In most cases, Model 2 with the largest contamination zone area produced the lowest DCGLs due to higher groundwater concentrations from reduced dilution and larger contaminated fractions from ingestion pathways. The results appear in Section 5.2.8 and were taken into account in establishing revised cleanup goals.

5.2.3 Streambed Sediment Conceptual Model

Figure 5-10 illustrates the conceptual model for development of streambed sediment DCGLs. Table 5-6 identifies the exposure pathways considered.

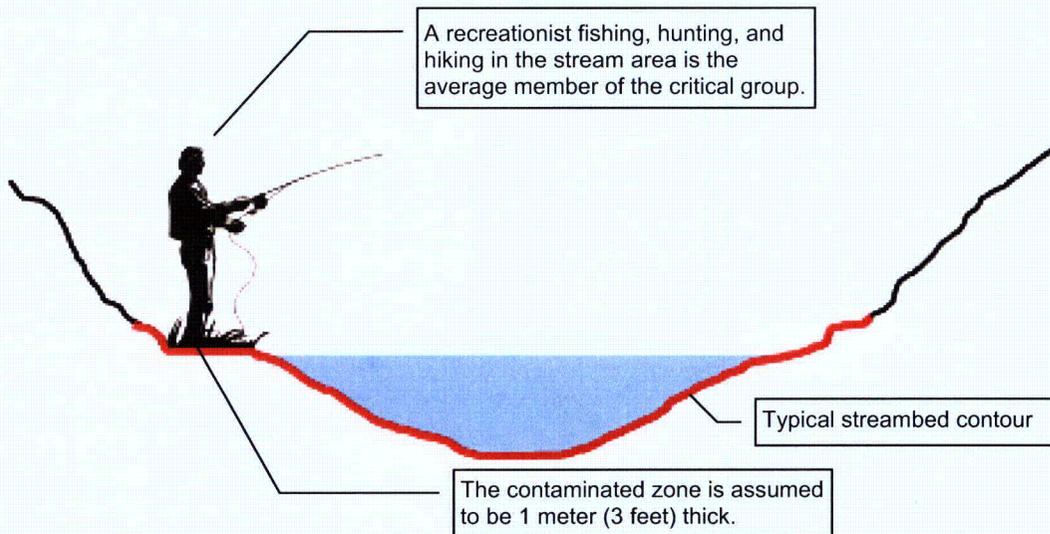


Figure 5-10. Conceptual Model for Streambed DCGLs Development

Table 5-6. Exposure Pathways for Streambed Sediment DCGL Development

Exposure Pathways	Active
External gamma radiation from contaminated sediment	Yes
Inhalation of airborne radioactivity from resuspended contaminated sediment	No ⁽¹⁾
Plant ingestion (produce impacted by soil and water sources)	No
Meat ingestion (venison impacted by soil and water sources)	Yes
Milk ingestion (impacted by soil and water sources)	No
Aquatic food ingestion (fish)	Yes
Ingestion of drinking water (from groundwater well)	No
Ingestion of drinking water (incidental from surface water)	Yes
Sediment ingestion (incidental during recreation)	Yes
Radon inhalation	No ⁽²⁾

NOTES: (1) Sediments adjacent to streambed have significant moisture content that inhibits their resuspension potential, which would minimize inhalation exposure. Additionally, vegetation along the streambed will likely preclude significant wind scour and subsequent inhalation. To confirm these conclusions, the model was revised to include the inhalation pathway as well as to make other minor refinements; these changes did not produce a significant difference in the results.

(2) The radon pathway is not considered because radon is primarily naturally occurring and neither radon nor its progeny are among the radionuclides of significant interest in dose modeling.