



10 CFR 50.90

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United States Nuclear Regulatory Commission
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Washington, DC 20555

Salem Generating Station – Units 1 and 2
Facility Operating License Nos. DPR-70 and DPR 75
NRC Docket Nos. 50-272 and 50-311

Subject: Supplement to License Amendment Request S09-06, Revision to Technical Specification 4.0.5, Surveillance Requirements for Inservice Inspection and Testing of ASME Code Components

Reference: (1) Letter from PSEG to NRC, "License Amendment Request (LAR) S09-06, Revision to Technical Specification 4.0.5, Surveillance Requirements for Inservice Inspection and Testing of ASME Code Components," dated September 23, 2009

In Reference 1, PSEG Nuclear LLC (PSEG) submitted a license amendment request for the facility operating licenses listed above. The request would modify Technical Specification (TS) TS 4.0.5 to be consistent with NRC-approved Technical Specification Task Force (TSTF) Travelers TSTF-479, Revision 0, "Changes to Reflect Revision of 10 CFR 50.55a," TSTF-497, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," and NUREG-1431, "Standard Technical Specifications, Westinghouse Plants, Revision 3.0."

Subsequent to submittal of Reference 1, PSEG discovered that one TS mark-up page was inadvertently omitted from the submittal. Specifically Salem Unit 1 TS page 3/4 4-32 (TS 3/4.4.10, Reactor Coolant System Structural Integrity) was omitted. A reference to TS 4.0.5 on this omitted page should have been marked up to reflect the proposed change; the marked up page is included as an attachment to this letter.

PSEG has reviewed the information supporting a finding of no significant hazards consideration that was provided to the NRC in Reference 1. The additional information provided in this letter does not affect the bases for concluding that the proposed license amendment does not involve a significant hazards consideration. No regulatory commitments are contained in this submittal.

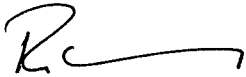
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If you have any questions or require additional information, please do not hesitate to contact Mr. Jeff Keenan at (856) 339-5429.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 12/20/09
(Date)

Sincerely,



Robert C. Braun
Site Vice President
Salem Generating Station

Attachments (1)

CC

S. Collins, Regional Administrator - NRC Region I
R. Ennis, Project Manager - USNRC
NRC Senior Resident Inspector - Salem
P. Mulligan, Manager IV, NJBNE
Commitment Coordinator - Salem
PSEG Corporate Commitment Manager

TECHNICAL SPECIFICATION PAGES WITH PROPOSED CHANGES

Facility Operating License DPR-70

Technical Specification

Page

3/4.4.10

3/4 4-32

REACTOR COOLANT SYSTEM

3.4.10 STRUCTURAL INTEGRITY

ASME CODE CLASS 1, 2 and 3 COMPONENTS

LIMITING CONDITION FOR OPERATION

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3.4.10.1 The structural integrity of ASME Code Class 1, 2 and 3 components shall be maintained in accordance with Specification 4.4.10.1.1.

APPLICABILITY: ALL MODES

ACTION:

- a. With the structural integrity of any ASME Code Class 1 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant system temperature more than 50°F above the minimum temperature required by NDT considerations.
- b. With the structural integrity of any ASME Code Class 2 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) prior to increasing the Reactor Coolant System temperature above 200°F.
- c. With the structural integrity of any ASME Code Class 3 component(s) not conforming to the above requirements, restore the structural integrity of the affected component(s) to within its limit or isolate the affected component(s) from service.

SURVEILLANCE REQUIREMENTS

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4.4.10.1.1 The structural integrity of ASME Code Class 1, 2 and 3 components shall be demonstrated:

- a. Per the requirements of ~~Specification 4.0.5~~ the Inservice Inspection Program, and
- b. Per the requirements of the augmented inservice inspection program specified in Specification 4.4.10.1.2.