



UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

OCT 13 1972

Docket No. 50-247

R. C. DeYoung, Assistant Director for Pressurized Water Reactors, L  
THRU: Karl Kniel, Chief, PWR Project Branch No. 2, L *KK*

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC., INDIAN POINT UNIT NO. 2  
(DOCKET NO. 50-247) MEETING ON SAFETY AND RELIEF VALVE INSTALLATION  
AND FUEL REPLACEMENT

Enclosed is a summary of the meeting regarding the evaluation of the  
safety and relief valve installation and the schedule for replacement  
of the fuel in the IP-2 core held with Consolidated Edison Company  
of New York, Inc., on October 4, 1972 in Bethesda, Maryland. An  
attendance list is also enclosed.

*M. A. McCoy*

M. A. McCoy  
PWR Project Branch No. 2  
Directorate of Licensing

Enclosures:

1. Meeting Summary
2. Attendance list

cc: w/encls.

R. S. Boyd

D. Skovholt

D. Knuth

R. Maccary

R. Tedesco

H. Denton

PWR Branch Chiefs

R. W. Klecker

M. Rosen

RO (3)

M. McCoy

L. L. Beratan

S. Hou

D. Lange

Docket file ✓

PWR-2 Reading

RP Reading

M. Service

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*Memo*

ENCLOSURE NO. 1

SUMMARY OF MEETING

SAFETY AND RELIEF VALVE INSTALLATION AND  
REPLACEMENT OF FUEL  
INDIAN POINT UNIT NO. 2

SUMMARY

A meeting was held in Bethesda on October 4, 1972, with the Consolidated Edison Company of New York, Inc., and representatives of Westinghouse Electric Corporation and United Engineers and Constructors to discuss the evaluation of the safety and relief valve installation and the schedule for replacement of fuel in the IP-2 core. The purpose of the meeting was to obtain additional information necessary for our evaluation of the applicant's report "Summary Report of Safety and Relief Valve Installation and Re-Analysis for ASME Class 1 and Class 2 Systems in Indian Point Unit No. 2" dated July 13, 1972. In addition, we were given the applicant's proposed mechanical design parameters for the modified fuel and a projected schedule for fuel loading and criticality.

DISCUSSION

Valve Installation

As a result of the review of the report identified above, it was determined that additional information was necessary to complete the evaluation. At this meeting, the applicant discussed a further description of the forcing functions applied to the piping systems to determine the mechanical response, a physical description of a typical piping modification, a description of the mathematical model used and a summary of stresses obtained for typical open and closed systems, a description of the analysis used to obtain the dynamic load factor for open systems, and a description of the analytical parameters for each computer program referenced in the report.

Procedures for documentation of this information were discussed. We intend to transmit a formal request for additional information.

Fuel

Con-Ed is in the process of reconstructing the IP-2 fuel as a result of its decision to convert to prepressurized fuel; new uranium at a higher densification will be used in core regions 2 and 3. The fuel rods in all regions will be prepressurized to 450 psig (cold). The new mechanical design parameters are tabulated as follows:

INDIAN POINT 2

PRESSURIZED FUEL

CORE MECHANICAL DESIGN PARAMETERS

Pellet density (% of theoretical)

Region 1	94	(same)
Region 2	95	
Region 3	95	

Feed enrichment (w/o)

Region 1	2.2	(same)
Region 2	2.8	
Region 3	3.3	

Pellet diameter, in.

Region 1	0.3669	(same)
Region 2	0.3659	
Region 3	0.3659	

Active fuel height, in.

Region 1	144.0	(same)
Region 2	142.0	
Region 3	142.0	

Redensification of the fuel pellets results in an increase in the total fuel weight to 220,000 pounds of  $UO_2$  and an increase in the total assembly weight to 279,000 pounds. The applicant stated that the number of burnable poison rods would be increased from 1160 to 1412.

Appropriate changes in the Design Features Chapter of the Technical Specifications will be made to reflect the new physical description of the core.

The applicant contemplates the following schedule for future review and fuel loading:

October 20, 1972	some fuel returned to site for storage
November 1, 1972	technical meeting for review of proposal
December 1, 1972	submittal of additional information for our review including accident analysis
January 15, 1972	fuel loading
February , 1972	criticality under potential testing licens

ENCLOSURE NO. 2

ATTENDANCE LIST

Consolidated Edison Company of New York, Inc.

C. W. Jackson  
W. J. Cahill, Jr.  
M. Silberstien  
E. P. Burke

Westinghouse

B. J. Garry  
Harry F. Clark, Jr.  
Irving M. Keyfitz  
Lester Berkowitz

LeBoeuf, Lamb

E. L. Cohen

UE&C

G. C. Duerr  
J. Dainora  
J. K. Lin

Atomic Energy Commission

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