

50-247

APR 3 1970

Peter A. Morris, Director
Division of Reactor Licensing

INDIAN POINT NUCLEAR GENERATING UNIT 2
ACRS REPORT SECTIONS

Enclosed are the four following sections of the ACRS report for the subject plant.

1. Reactor Vessel Material Surveillance Program
2. Engineered Safety Features and Reactor Coolant Leak Detection
3. Flywheel Integrity
4. Inservice Inspection

Original signed by
Edson G. Case, Director
Division of Reactor Standards

DRS:SEB:MBF

Enclosure:
ACRS Report Sections for
Indian Point Unit 2

cc w/encl:
R. S. Boyd, DRL
R. Maccary, DRS
D. Muller, DRL
A. Dromerick, DRS
K. Kniel, DRL
M. Fairtile

bcc:
Distribution:
Suppl Docket No. 50-247 ←
DRS Reading
DR Reading
SEB Reading

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OFFICE ▶	DRS:SEB	DRS:JED	DRS:AD/E	DRS:DR		
SURNAME ▶	MBFairtile:ewe	A. Dromerick	RMaccary	EGCase		
DATE ▶	4-1-70	4-1-70	3-70	4-1-70		

INDIAN POINT NUCLEAR GENERATING UNIT 2

ACRS REPORT SECTIONS

1. Reactor Vessel Material Surveillance Program

The estimated end-of-life fluence of 2.4×10^{19} nvt, taking into account that the power density of this core is less than that of other Westinghouse four-loop plants, is reasonable. The calculational model and the proposed material surveillance program are similar to other Westinghouse plants we have recently evaluated. We conclude the Indian Point 2 program is acceptable and consistent with previous acceptable PWR plant reviews.

2. Engineered Safety Features and Reactor Coolant Leak Detection

The systems are similar to those proposed for other Westinghouse plants we have recently evaluated. We conclude that the systems are acceptable for Indian Point 2.

3. Flywheel Integrity

The primary pump-motor flywheels used in this plant are identical to those used in other Westinghouse plants. These standard Westinghouse designed flywheels are fabricated of vacuum degassed A-533B steel. On the basis of previous evaluations, we conclude that the flywheel design is acceptable in view of the use of high grade material, extensive quality control measures, and preservice and inservice surveillance requirements.

4. Inservice Inspection

The program proposed by the applicant in the technical specifications is similar to that already approved for Ginna. While the program meets our requirements for plants constructed prior to the adoption of the

ASME Boiler and Pressure Vessel Code, Section XI, we are in our review of the Technical Specifications upgrading the program to the full potential of the accessibility inherent in the plant design. We conclude that the Indian Point 2 inservice inspection program is acceptable.



UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

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A handwritten signature in black ink, appearing to read "Edson G. Case", is located to the right of the typed name.

Edson G. Case, Director
Division of Reactor Standards

DRS:SEB:MBF

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