



Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, N.Y. 10003

October 16, 1980

Re: Indian Point Unit No. 2
Docket No. 50-247

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

In accordance with the Technical Specifications of Facility Operating License No. DPR-26, the following confirms notification to Mr. Ted Rebelowski of your office, by Mr. John Makepeace of Consolidated Edison, of a potential Reportable Occurrence LER-80-012/01. This event is the type defined in Technical Specification, Section 6.9.1.7.1 (b).

As a followup to Reportable Occurrence LER-80-011/01, Con Edison immediately initiated a review of plant procedures to determine if there were any additional areas where there might be a potential conflict between procedural steps and regulatory requirements. In the course of conducting this review, one such area was identified on October 16, 1980. Specifically, the Inservice Valve Test (PT-Q13) for the two parallel pneumatically operated diaphragm valves (Valve Nos. 876A and 876B) in the discharge line from the sodium hydroxide additive tank required that manually operated valve 1841 in the common discharge line upstream of the two parallel valves be closed when these valves are stroked. This precaution was taken to preclude the possibility of cross contamination of the sodium hydroxide additive tank and the refueling water storage tank during stroking of the two parallel discharge valves. The test procedure cautioned that valve 1841 must be opened within two (2) minutes if a phase B containment isolation signal is initiated.

Until we complete a full evaluation of this occurrence, Test PT-Q13 has been revised to delete the requirement for closing valve 1841 during testing of valves 876A and 876B.

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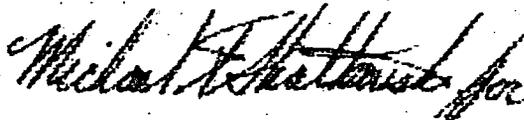
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We are continuing the procedure review discussed above and will immediately advise you by telephone (with written confirmation within twenty-four hours) should any further discrepancies be uncovered. This review is scheduled to be completed by December 15, 1980. At that time, we will provide the NRC with a summary report of our findings.

Consolidated Edison believes that this report also satisfies the requirements of 10 CFR Part 21.

Very truly yours,



William A. Monti, Manager
Nuclear Power Generation
Department
Indian Point Station
Buchanan, N.Y. 10511

JMM/mcs

cc: Mr. William McDonald, Director (2 copies)
Office of Management Information and Program Control
c/o Distribution Services Branch, DDC, ADM
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. T. Rebelowski, Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 38
Buchanan, N.Y. 10511

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | _____ (2) | _____ (3) | _____ (4) | _____ (5)
7 8 9 | LICENSEE CODE 14 15 | LICENSE NUMBER 25 26 | LICENSE TYPE 30 | 57 CAT 58

CON'T
0 1 | _____ (6) | 510-247 | _____ (7) | _____ (8) | 101680 | _____ (9)
7 8 | REPORT SOURCE 60 61 | DOCKET NUMBER 68 69 | EVENT DATE 74 75 | REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | _____
0 3 | _____
0 4 | _____
0 5 | _____
0 6 | _____
0 7 | _____
0 8 | _____

See Letter

0 9 | _____ (11) | _____ (12) | _____ (13) | _____ (14) | _____ (15) | _____ (16)
7 8 9 | SYSTEM CODE 9 10 | CAUSE CODE 11 | CAUSE SUBCODE 12 13 | COMPONENT CODE 18 | COMP. SUBCODE 19 | VALVE SUBCODE 20

(17) LER/RO REPORT NUMBER | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | REVISION NO.
21 22 23 24 25 26 27 28 29 30 31 32

ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS | ATTACHMENT SUBMITTED | NPRD-4 FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | _____
1 1 | _____
1 2 | _____
1 3 | _____
1 4 | _____

1 5 | _____ (28) | _____ (29) | _____ (30) | _____ (31) | _____ (32)
7 8 9 | FACILITY STATUS 10 | % POWER 11 | OTHER STATUS 12 13 | METHOD OF DISCOVERY 14 | DISCOVERY DESCRIPTION 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 6 | _____ (33) | _____ (34) | _____ (35) | _____ (36)
7 8 9 | ACTIVITY RELEASED 10 | CONTENT OF RELEASE 11 | AMOUNT OF ACTIVITY 12 13 | LOCATION OF RELEASE 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 7 | _____ (37) | _____ (38) | _____ (39)
7 8 9 | PERSONNEL EXPOSURES NUMBER 10 | TYPE 11 | DESCRIPTION 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 8 | _____ (40) | _____ (41)
7 8 9 | PERSONNEL INJURIES NUMBER 10 | DESCRIPTION 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 9 | _____ (42) | _____ (43)
7 8 9 | LOSS OF OR DAMAGE TO FACILITY TYPE 10 | DESCRIPTION 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

2 0 | _____ (44) | _____ (45)
7 8 9 | PUBLICITY ISSUED 10 | DESCRIPTION 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

NAME OF PREPARER _____

PHONE: _____

NRC USE ONLY