

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 N Y I P S 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 57 CAT 58

CON'T

0 1 REPORT SOURCE L 0 5 0 0 0 2 4 7 1 2 0 2 7 9 1 2 2 8 7 9 9
7 8 60 61 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During power escalation following a maintenance outage, reactor
0 3 coolant leakage into the pressurizer relief tank was calculated to be
0 4 approximately 11.6 GPM. The Unit was shutdown in accordance with the
0 5 requirements of T.S. 3.1.F.2. Subsequent investigation revealed the
0 6 source of the leakage to be letdown relief valve No. 203.
0 7

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
P C 11 F 12 B 13 V A L V E X 14 X 15 B 16
7 8 9 10 11 12 13 18 19 20

(17) LER/RO
REPORT NUMBEREVENT YEAR
7 9SEQUENTIAL
REPORT NO.
0 2 4OCCURRENCE
CODE
0 3REPORT
TYPE
LREVISION
NO.
0ACTION
TAKEN
A 18FUTURE
ACTION
Z 19EFFECT
ON PLANT
A 20SHUTDOWN
METHOD
A 21HOURS
0 1 5 6ATTACHMENT
SUBMITTED
Y 23NPRD-4
FORM SUB.
N 24PRIME COMP.
SUPPLIER
N 25COMPONENT
MANUFACTURER
C 7 1 0 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Excessive leakage through a Crosby style 2XJX3, s.s. relief valve,
1 1 Dwg. No. H-51680, was caused by degradation of the valve's seating
1 2 surface. The valve's nozzle (seat) and disc insert were replaced and
1 3 the valve retested satisfactorily.
1 4

1 5 FACILITY STATUS C 28 % POWER 0 9 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Control Room Instrumentation 32
7 8 9 10 12 13 44 45 46 80

1 6 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
7 8 9 10 11 44 45 80

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39
7 8 9 10 11 12 13 44 45 80

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
7 8 9 10 11 12 44 45 80

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
7 8 9 10 44 45 80

2 0 PUBLICITY ISSUED N 44 DESCRIPTION NA 45
7 8 9 10 44 45 80

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NRC USE ONLY

ATTACHMENT I

Docket No. 50-247

Consolidated Edison Co. of N.Y., Inc.

LER-79-024/03 L-O

Indian Point Unit No. 2

During power escalation on December 2, 1979 following a maintenance outage, leakage of reactor coolant into the pressurizer relief tank was indicated by increasing level, temperature and pressure within the tank and by increased temperature within the relief valve discharge header to the tank. Calculations indicated a leakage rate of 11.6 gallons per minute. In accordance with the requirements of Technical Specification 3.1.F.2, the Unit was shutdown. During the course of the shutdown the leakage rate decreased to within the Technical Specification allowable leak rate.

As a precautionary measure, the plant was placed in the cold shutdown condition to allow inspection and testing of possible leakage sources. Two possible sources of this leakage existed, the pressurizer safety valves and the letdown relief valve. Investigation of these potential leakage sources revealed the following:

Upon inspection, letdown relief valve No. 203 was found to have cuts in its seating surface. The valve's nozzle (seat) and disc insert were replaced in like and kind. The valve was then leak tested, setpoint verified three times and then leak tested again. All tests gave satisfactory results and the valve was reinstalled in the letdown line.

Pressurizer safety valves (Nos. PCV-464, 466 and 468) were leak tested and setpoint was then verified three times. These tests indicated the valves were leak tight and were operating at their proper setpoint. A leak test following this, however, indicated some leakage through the valve seats. The safety valve seats were lapped as a precautionary measure and the leakage and setpoint tests repeated. All tests gave satisfactory results and the valves were reinstalled in their designated positions.

At all times the reactor coolant leakage rate was well within the capability of the makeup system and resulted in no difficulties concerning normal pressurizer level and pressure control. The leakage was contained within closed systems and caused no uncontrolled release of radioactive material.