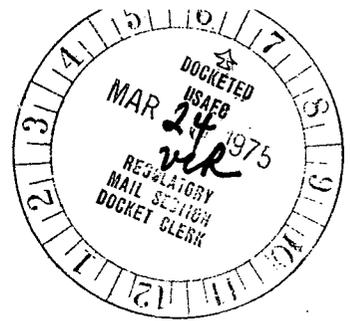


REGULATORY DOCKET FILE COPY



Consolidated Edison Company of New York, Inc.  
4 Irving Place, New York, N Y 10003



March 17, 1975

Re: Indian Point Unit No. 2  
AEC Docket No. 50-247  
A.O. 5-2-2

Mr. Edson G. Case, Acting Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20545

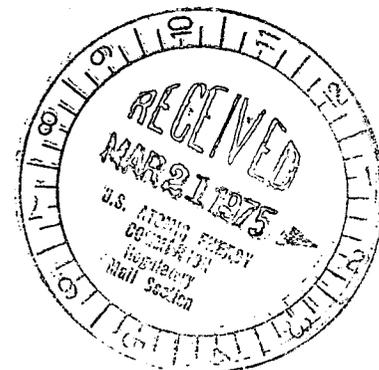
Dear Mr. Case:

In accordance with the requirements of the Technical Specifications to Facility Operating License DPR-26, the attached report of an Abnormal Occurrence is submitted.

*Walter Stein*

Walter Stein, Manager  
Nuclear Power Generation

Copy to: Mr. James P. O'Reilly  
Regulatory Operations



8111090349 750317  
PDR ADOCK 05000247  
S PDR

3071

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL  
(TEMPORARY FORM)

CONTROL NO: 3071

FILE: INCIDENT REPORT FILE

|  |               |                        |                       |                   |                                |     |            |
|--|---------------|------------------------|-----------------------|-------------------|--------------------------------|-----|------------|
| FROM: Con Edison<br>New York, N.Y.<br>Walter Stein |               | DATE OF DOC<br>3-17-75 | DATE REC'D<br>3-21-75 | LTR<br>XX         | TWX                            | RPT | OTHER      |
| TO: Edson G. Case                                  |               | ORIG<br>1 signed       | CC                    | OTHER             | SENT AEC PDR<br>SENT LOCAL PDR |     | XXX<br>XXX |
| CLASS  | UNCLASS<br>XX | PROP INFO              | INPUT                 | NO CYS REC'D<br>1 | DOCKET NO:<br>50-247           |     |            |

DESCRIPTION:  
Ltr. reporting Abn. Occurr. #50-247/5-2-2  
trans. the following...

ENCLOSURES:  
" Reactor coolant Flow analog channel Function-  
al Test" found bistable FC=446 exceeded it's  
safety system setting.

**ACKNOWLEDGED**  
**DO NOT REMOVE**

PLANT NAME: Indian Point #2

FOR ACTION/INFORMATION

VCR 3-24-75

|                         |                            |                             |                        |
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|---|---|---|--|--|

EXTERNAL DISTRIBUTION

|                                       |                              |   |
|---------------------------------------|------------------------------|---|
| - 1 - LOCAL PDR <u>Montrose, N.Y.</u> | - NATIONAL LABS              | 1 - PDR-SAN/LA/NY                       |
| - 1 - TIC (ABERNATHY) (1)(2)(10)      | - W. PENNINGTON, Rm E-201 GT | 1 - BROOKHAVEN NAT LAB                  |
| - 1 - NSIC (BUCHANAN)                 | - CONSULTANTS                | 1 - G. ULRIKSON, ORNL                   |
| - 1 - ASLB                            | NEWMARK/BLUME/AGBABIAN       | 1 - AGMED (RUTH GUSSMAN)<br>Rm B-127 GT |
| - 1 - Newton Anderson                 |                              | 1 - J. D. RUNKLES, Rm E-201<br>GT       |
| - 5 - ACRS SENT TO LIC ASST           |                              |   |
| ** SEND ONLY TEN DAY REPORTS          |                              |   |

1. Report Number: 50-247/5-2-2
- 2a. Report Date: March 17, 1975
- 2b. Occurrence Date: March 7, 1975
3. Facility: Indian Point Unit No. 2
4. Identification of Occurrence:

This abnormal occurrence was the type identified by Technical Specification 1.8.a where a protective instrumentation setting was found in excess of a limiting safety system setting established in the Technical Specifications.

5. Conditions Prior to Occurrence:

At the time of the occurrence, the unit was in the cold shutdown condition for a planned three week outage which began on February 28, 1975.

6. Description of Occurrence:

On March 7, 1975, during the performance of periodic surveillance test, PT-M3, "Reactor Coolant Flow Analog Channel Functional Test" it was found that bistable FC-446 exceeded its limiting safety system setting.

FC-446 was found to be set 11.42 milliamps below the limiting setting. This "as found" deviation resulted in low reactor coolant flow setting of approximately 67% of normal indicated loop flow instead of the required equal to or greater than 90%. All other bistables of this logic were found to be set correctly.

7. Designation of Apparent Cause of Occurrence:

The apparent cause of the occurrence has been determined to be a malfunctioning power supply board in the bistable.

The bistable was removed from the logic cabinet and bench tested to determine the cause. As a result of this testing, it was found that the reference power supply was operating at 4.7 volts instead of the specified 5.2V D.C.  $\pm$  0.25V D.C.

8. Analysis of Occurrence:

Bistable FC-446 provides input to the loop No. 24 low reactor coolant flow reactor trip logic. This particular bistable is one channel in a logic which actuates when low reactor coolant flow is sensed in any two out of three channels. The remaining two channels functioned correctly during performance of the periodic test and, therefore, the above trip would have actuated at the correct setpoint.

In addition, the reactor coolant pump underfrequency and undervoltage reactor trips were also available for protection if required.

In light of the above, the safety implications of this occurrence are considered to be slight.

9. Corrective Action:

The power supply panel in bistable FC-446 was replaced and the bistable tested satisfactorily.

10. Failure Data:

Bistable FC-446

Foxboro Electronic  
Model No. 63S-AR-OAHA  
Serial No. 1859528

Power Supply Board

Part No. N116PF

A previous failure of a power supply board was reported in A.O. 4-2-22 dated August 2, 1974.

11. Notification:

An initial report of this occurrence was provided the Region 1 Office of Inspection and Enforcement by telephone on March 7, 1975, followed by letter dated March 10, 1975.