



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 22, 2010

Mr. David J. Bannister
Vice President and CNO
Omaha Public Power District
Fort Calhoun Station
444 South 16th St. Mall
Omaha, NE 68102-2247

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 – CORRECTION TO AMENDMENT
NO. 263 RE: REQUEST TO ADD STEAM GENERATOR BLOWDOWN ISOLATION
REQUIREMENTS TO TECHNICAL SPECIFICATIONS (TAC NO. ME0596)

Dear Mr. Bannister:

On October 9, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML092540591), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 263 to Renewed Facility Operating License No. DPR-40 for the Fort Calhoun Station, Unit No. 1. The amendment consisted of changes to the Technical Specifications (TSs) in response to your application dated January 30, 2009, as supplemented by letters dated June 30 and August 28, 2009 (ADAMS Accession Nos. ML090340536, ML091830041, and ML092440295, respectively).

Amendment No. 263 added steam generator blowdown isolation operability and surveillance requirements to the TSs. These requirements will be needed as a result of a plant modification to add an interlock to automatically isolate steam generator blowdown following a reactor trip. In addition, the amendment modified the remedial actions for channels that currently contain a key-operated bypass switch by deleting the term “key-operated” so that channels with a bypass switch that are not key-operated may be bypassed as well.

On June 30, 2008, the NRC staff issued Amendment No. 257 (ADAMS Accession No. ML081790191), which renumbered most of the pages in TS 3.1. Upon implementation of Amendment No. 257, what had been TS 3.1 – Page 13 became TS 3.1 – Page 14. Due to an administrative error, the NRC issued Amendment No. 263 which revised TS 3.1 – Page 14 improperly as TS 3.1 – Page 13 thereby creating two TS pages having the same page number, TS 3.1 – Page 13.

This administrative error did not impact Amendment No. 263 for FCS. The error does not change the NRC staff conclusions regarding Amendment No. 263 for FCS. Enclosed are corrected versions of TS 3.1 – Page 13 (Amendment 257) and TS 3.1 – Page 14 (Amendment 263). Please discard the associated pages from the previous amendments and replace them with the enclosed pages.

D. Bannister

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If you have any questions, please contact me at 301-415-1377 or lynnea.wilkins@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "Lynnea Wilkins". The signature is fluid and cursive, with the first name "Lynnea" written in a larger, more prominent script than the last name "Wilkins".

Lynnea Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures:
As stated

cc w/encl: Distribution via Listserv

ENCLOSURES

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT 1

DOCKET NO. 50-285

Corrected Technical Specification Page 3.1 – Page 13 (Amendment 257)

AND

Corrected Technical Specification Page 3.1 – Page 14 (Amendment 263)

TECHNICAL SPECIFICATIONS

TABLE 3-2 (continued)

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TESTING OF ENGINEERED SAFETY FEATURES, INSTRUMENTATION AND CONTROLS

<u>Channel Description</u>	<u>Surveillance Function</u>	<u>Frequency</u>	<u>Surveillance Method</u>
18. SIRW Tank Temperature	a. Check	D ⁽⁶⁾	a. Verify that temperature is within limits.
	b. Test	R	b. Measure temperature of SIRW tank with standard laboratory instruments.
19. Manual Recirculation Actuation	a. Test	R	a. CHANNEL FUNCTIONAL TEST
20. Recirculation Actuation Logic	a. Test	Q	a. CHANNEL FUNCTIONAL TEST
	b. Test	R ⁽⁷⁾	b. CHANNEL FUNCTIONAL TEST
*21. 4.16 KV Emergency Bus Low Voltage (Loss of Voltage and Degraded Voltage) Actuation Logic	a. Check	S	a. Verify voltage readings are above alarm initiation on degraded voltage level - supervisory lights "on".
	b. Test	Q	b. CHANNEL FUNCTIONAL TEST (Undervoltage relay)
	c. Calibrate	R	c. CHANNEL CALIBRATION
22. Manual Emergency Off-site Power Low Trip Actuation	a. Test	R	a. CHANNEL FUNCTIONAL TEST

TECHNICAL SPECIFICATIONS

TABLE 3-2 (continued)

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TESTING OF ENGINEERED SAFETY FEATURES, INSTRUMENTATION AND CONTROLS

<u>Channel Description</u>	<u>Surveillance Function</u>	<u>Frequency</u>	<u>Surveillance Method</u>
23. Auxiliary Feedwater	a. Check: 1) Steam Generator Water Level Low (Wide Range) 2) Steam Generator Pressure Low	S	a. 1) CHANNEL CHECK 2) CHANNEL CHECK
	b. Test: 1) Actuation Logic	QR ⁽⁷⁾	b. 1) CHANNEL FUNCTIONAL TEST
	c. Calibrate: 1) Steam Generator Water Level Low (Wide Range) 2) Steam Generator Pressure Low 3) Steam Generator Differential Pressure High	R	c. 1) CHANNEL CALIBRATION 2) CHANNEL CALIBRATION 3) CHANNEL CALIBRATION
24. Manual Auxiliary Feedwater Actuation	a. Test	R	a. CHANNEL FUNCTIONAL TEST
25. Manual Steam Generator Blowdown Isolation	a. Test	R	a. CHANNEL FUNCTIONAL TEST
26. Automatic Steam Generator Blowdown Isolation	a. Test	R	a. CHANNEL FUNCTIONAL TEST

- NOTES:** (1) Not required unless pressurizer pressure is above 1700 psia.
(2) CRHS monitors are the containment atmosphere gaseous radiation monitor and the Auxiliary Building Exhaust Stack gaseous radiation monitor.
(3) Not required unless steam generator pressure is above 600 psia.
(4) QP - Quarterly during designated modes and prior to taking the reactor critical if not completed within the previous 92 days (not applicable to a fast trip recovery).
(5) Not required to be done on a SIT with inoperable level and/or pressure instrumentation.
(6) Not required when outside ambient air temperature is greater than 50°F and less than 105°F.
(7) Tests backup channels such as derived circuits and equipment that cannot be tested when the plant is at power.
(8) SGLS is required for containment spray pump actuation only. SGLS lockout relays are not actuated for this test.

D. Bannister

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If you have any questions, please contact me at 301-415-1377 or lynnea.wilkins@nrc.gov.

Sincerely,

/RA/

Lynnea Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-285

Enclosures:
As stated

cc w/encl: Distribution via Listserv

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ADAMS Accession No. ML093561119

OFFICE	NRR/LPL4/PM	NRR/LPL4/PM	NRR/LPL4/LA	NRR/LPL4/BC	NRR/LPL4/PM
NAME	LWilkins	AWang	JBurkhardt	MMarkley	LWilkins
DATE	1/19/10	1/20/10	1/19/10	1/22/10	1/22/10

OFFICIAL AGENCY RECORD