

LaSalle Generating Station  
2601 North 21st Road  
Marseilles, IL 61341-9757

www.exeloncorp.com

December 11, 2009

10 CFR 50.73

United States Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D.C. 20555

LaSalle County Station, Unit 2  
Facility Operating License No. NPF 18  
NRC Docket No. 50-374

Subject: Licensee Event Report

In accordance with 10 CFR 50.73 (a)(2)(i)(A), Exelon Generation Company, (EGC), LLC, is submitting Licensee Event Report Number 09-002-00, Docket No. 050-374.

Should you have any questions concerning this letter, please contact Mr. Terrence W. Simpkin, Regulatory Assurance Manager, at (815) 415-2800.

Respectfully,



David P. Rhoades  
Plant Manager  
LaSalle County Station

Attachment: Licensee Event Report

cc: Regional Administrator - NRC Region III  
NRC Senior Resident Inspector - LaSalle County Station

1E22  
NRK

### LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

<b>1. FACILITY NAME</b> LaSalle County Station Unit 2	<b>2. DOCKET NUMBER</b> 05000374	<b>3. PAGE</b> 1 OF 4
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**4. TITLE**  
Completed Plant Shutdown Due to Unidentified Leakage in Excess of Technical Specification Limits

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	12	2009	2009	002	00	12	11	2009	FACILITY NAME	DOCKET NUMBER

<b>9. OPERATING MODE</b>  1	<b>11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §:</b> (Check all that apply)							
<b>10. POWER LEVEL</b>  100	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)				
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)				
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)				
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)				
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)				
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)				
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)					
<input type="checkbox"/> 20.2203(a)(2)(v)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER					
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A					

**12. LICENSEE CONTACT FOR THIS LER**

<b>FACILITY NAME</b> Barb Houston, Maintenance Support Manager	<b>TELEPHONE NUMBER (Include Area Code)</b> 815-415-2501
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**13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT**

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX
D	AD	ISV	V085	Y					

<b>14. SUPPLEMENTAL REPORT EXPECTED</b> <input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	<b>15. EXPECTED SUBMISSION DATE</b>	MONTH	DAY	YEAR
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**ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)**

On October 11, 2009, at 2333 CDT, unidentified leakage on LaSalle Unit 2 exceeded the Technical Specification (TS) Limiting Condition for Operation 3.4.5.d limit of less than or equal to a two gallon per minute increase in the previous 24 hour period while in Mode 1. Required Action (RA) B.1 requires that the increase in unidentified leakage be reduced below the limit within four hours. If RA B.1 can not be met, RA C.1 and C.2 require that the unit be in Mode 3 within 12 hours and Mode 4 within 36 hours.

A plant shutdown was commenced at 0300 hours on October 12, 2009, and Mode 4 was reached at 0005 hours on October 13, 2009.

The unidentified leakage was determined to be from a packing leak on the Inboard Main Steam Isolation Valve Drain Line Isolation Valve (2B21-F016). The packing leak was repaired and the unit was restarted on October 14, 2009. Corrective actions to prevent recurrence include training maintenance personnel on the different types of gaskets used on pressure seal valves, and revisions to the subject maintenance procedures.

**LICENSEE EVENT REPORT (LER)  
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE	
LaSalle County Station Unit 2	05000374	YEAR	SEQUENTIAL NUMBER	REV NO.	2	OF 4
		2009	- 002 -	00		

**NARRATIVE**

**A. PLANT AND SYSTEM IDENTIFICATION**

General Electric Boiling Water Reactor, 3489 Megawatts Thermal Rated Core Power

**CONDITION PRIOR TO EVENT**

Unit(s): 2	Event Date: 10/12/2009	Event Time: 1405
Reactor Mode(s): 1	Power Level(s): 100	
Mode(s) Name: Power Operation		

**B. DESCRIPTION OF EVENT**

On October 11, 2009, at 2333 CDT, unidentified leakage on LaSalle Unit 2 exceeded the Technical Specification (TS) Limiting Condition for Operation (LCO) 3.4.5.d limit of less than or equal to a two gallon per minute (gpm) increase in the previous 24 hour period while in Mode 1. Required Action (RA) B.1 requires that the increase in unidentified leakage be reduced below the limit within four hours. If RA B.1 can not be met, RA C.1 and C.2 require that the unit be in Mode 3 within 12 hours and Mode 4 within 36 hours.

At 0300 hours on October 12, 2009, a plant shutdown was commenced. At 0333 hours, unidentified leakage was still in excess of the TS LCO 3.4.5.d limit. The unit entered Mode 3 at 1405 hours on October 12, and reached Mode 4 at 0005 hours on October 13, 2009.

The unidentified leakage was determined to be from a packing leak on the Inboard Main Steam Isolation Valve (MSIV) Drain Line Isolation Valve (2B21-F016) (NB)[AD]. The packing leak was repaired and the unit was restarted on October 14, returning to full power on October 17, 2009.

The event was reportable under 10 CFR50.72(b)(2)(i) as an initiation of a nuclear plant shutdown required by TS. An ENS call (EN# 45426) was made on October 12, 2009, at 0433 CDT. The event is also reportable under 10 CFR 50.73(a)(2)(i)(A), as the completion of a nuclear plant shutdown required by TS.

**C. CAUSE OF EVENT**

The cause of the event was a packing leak on the Inboard MSIV Drain Line Isolation Valve (2B21-F016). The leak occurred when the 2B21-F016 valve was stroked closed and then full open during a 2B Reactor Protection System (RPS) bus transfer.

A root cause investigation determined that there was a body-to-bonnet misalignment at the completion of the 2005 valve installation that was not noticed by the workers. The misalignment was due, in part, to a lack of clarity and specificity in the work package and vendor manual instructions. Additionally, the misalignment was masked by the fact that a graphite gasket was used, which was more malleable than the iron gaskets commonly used, and did not leak immediately following the work despite the misalignment.

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE	
LaSalle County Station Unit 2	05000374	YEAR	SEQUENTIAL NUMBER	REV NO.	3	OF 4
		2009	- 002	- 00		

**NARRATIVE**

**D. SAFETY ANALYSIS**

The safety significance of the event was minimal. Makeup capability was adequate to compensate for the leak, which was approximately 2.3 gpm. All Emergency Core Cooling Systems (ECCS) were operable and capable of fulfilling their intended safety function during the period of excessive leakage.

The event did not constitute a safety system functional failure.

**E. CORRECTIVE ACTIONS**

Corrective Actions:

- The 2B21-F016 valve was repacked and repaired.

Corrective Action to Prevent Recurrence:

- Mechanical Maintenance personnel and supervisors will be trained on the different types of gaskets used on pressure seal valves and their installation requirements.
- A maintenance procedure will be developed specifically for the Inboard and Outboard MSIV Drain Line Isolation Valves (1/2B21-F016 and 1/2B21-F019) to incorporate vendor manual instructions, including ensuring proper alignment by installing the valve operator in order to pull up the inner bonnet prior to tightening.
- Other pressure seal valve maintenance procedures will be revised as necessary to incorporate vendor manual instructions for ensuring proper body-to-body alignment.

**F. PREVIOUS OCCURENCES**

A document review covering the previous three years found no previous occurrences of a plant shutdown required by excessive unidentified leakage.

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE		
LaSalle County Station Unit 2	05000374	YEAR	SEQUENTIAL NUMBER	REV NO.	4	OF	4
		2009	- 002	- 00			

**NARRATIVE**

**G. COMPONENT FAILURE DATA**

Velan Pressure Seal Gate Valve (Forged) 3"