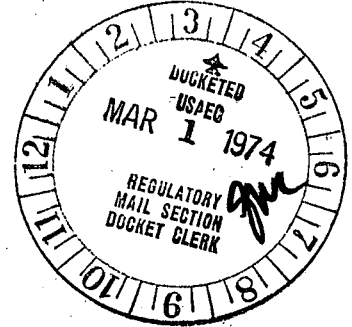


February 26, 1974

Re: Indian Point Unit No. 2
AEC Docket No. 50-247
Operating License DPR-26
A.O. 4-2-9

Mr. James P. O'Reilly, Director
Regulatory Operations, Region I
U. S. Atomic Energy Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406



Dear Mr. O'Reilly:

In accordance with the requirements of Section 6.12.2(a) of the Technical Specifications of Facility Operating License No. DPR-26, the following report is submitted:

In the course of performing periodic surveillance test PT-M11, "Steam Line Pressure Analog Channel Functional Test" on February 22, 1974, an inadvertent safety injection signal was generated which, by design, caused the accumulator discharge stop valves to open. At the time of the occurrence, the reactor was in the cold shutdown condition with the Residual Heat Removal System in service and a reactor coolant pressure and temperature of 150 psig and 115°F respectively. Since the reactor coolant system was being operated in the solid mode, opening of the accumulator valves pressurized the system to about 560 psig which was slightly in excess of the Technical Specification limit of 500 psig for indicated temperatures at or below 220°F. The pressure was promptly reduced below the 500 psig limit by operator action.

There was no damage incurred to any system or component as a result of the pressure transient of this magnitude nor was there any reason to expect any.

Mr. Anthony Fasano of your office was informed of this occurrence by Mr. John M. Makepeace on February 22, 1974.

Very truly yours,

Warren R. Cobean, Jr.

Warren R. Cobean, Jr. Manager
Nuclear Power Generation Depart.

cc: John F. O'Leary

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