Regulatory Docket File

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Re Indian Point Unit No. AEC Docket No. 50-247 Facility Operating License No. DPR-26

Mr. John F. O'Leary, Director Directorate of Licensing U. S. Atomic Energy Commission Washington, D. C. 20545

Dear Mr. O'Leary

Enclosed is a report entitled, "Report on Results of Test Program Following Plant Revisions Affecting Feedwater Piping", dated March 12, 1974, concerning the incident which occurred at Indian Point Unit No. 2 on November 13, 1973, and the subsequent testing programs.

In our January 14, 1974 meeting with the Regulatory Staff in Bethesda, Maryland, we discussed our intention of performing a test program to assure ourselves that the modifications made to the Feedwater System would be adequate to avoid a water-hammer shock such as occurred on November 13, 1973. During the performance of these tests, after restarting the plant, indications of waterhammer were observed, and data were recorded on the instrumentation specially provided to monitor waterhammer which indicated that the observed waterhammer occurred at high rates of auxiliary feedwater flow during recovery from low level trips. Thus, waterhammer associated with excessive flow rate during level recovery could be minimized by limitation of flow rate either by administrative control or modification of the regulating system.

However, it was determined that a more basic solution could be achieved by minor modifications to the steam generator internals, and it was decided to accomplish these modifications during the current outage. The enclosed report describes these alternatives and outlines a startup test program that has been developed to further assure that the problem has been solved.

The incidents, results of the ensuing investigation and the modifications under way have been reviewed by the Station Nuclear Safety Committee and the Nuclear Facilities Safety



Mr. John F. O'Leary -2-March 12, 1974 Atomic Energy Commission Indian Point Unit No. 2 Re AEC Docket No. 50-247 Facility Operating License No. DPR-26 Committee. Both Committees determined that there were no unreviewed safety questions or Technical Specification changes involved. Very truly yours William J. Cahill, Jr. enc. Vice President md