

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

FACILITY MANAGEMENT AND TECHNICAL SUPPORT

6.2.1 The organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling. This individual shall have no other concurrent responsibilities during this operation.

6.2.3 Any change in the organizations shown on Figures 6.2-1 and 6.2-2 shall be reported in accordance with section 6.9.2.e and the implementation of any such change, which is not inconsistent with other requirements of these technical specifications, shall not be deemed a violation.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

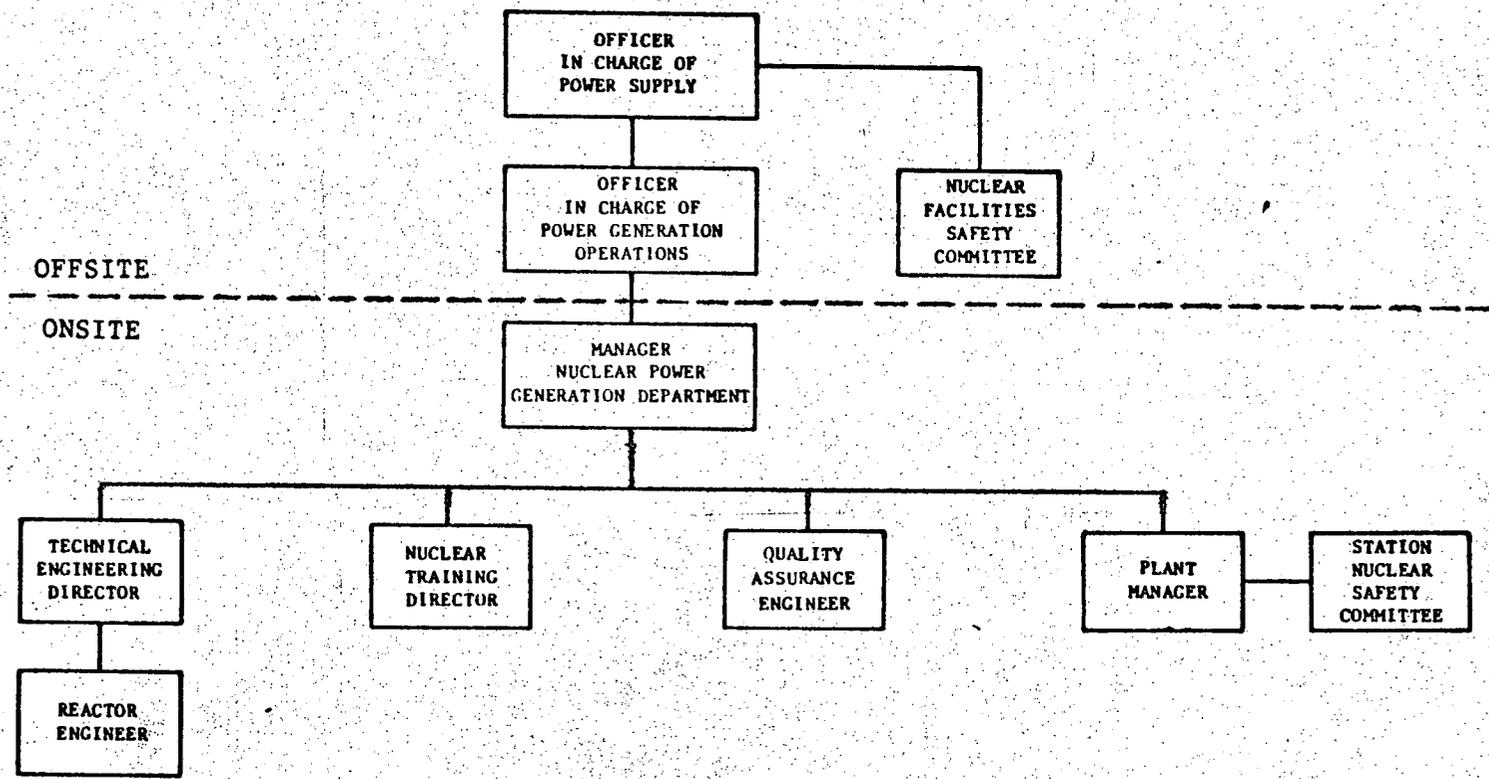


Figure 6.2-1 Facility Management and Technical Support Organization

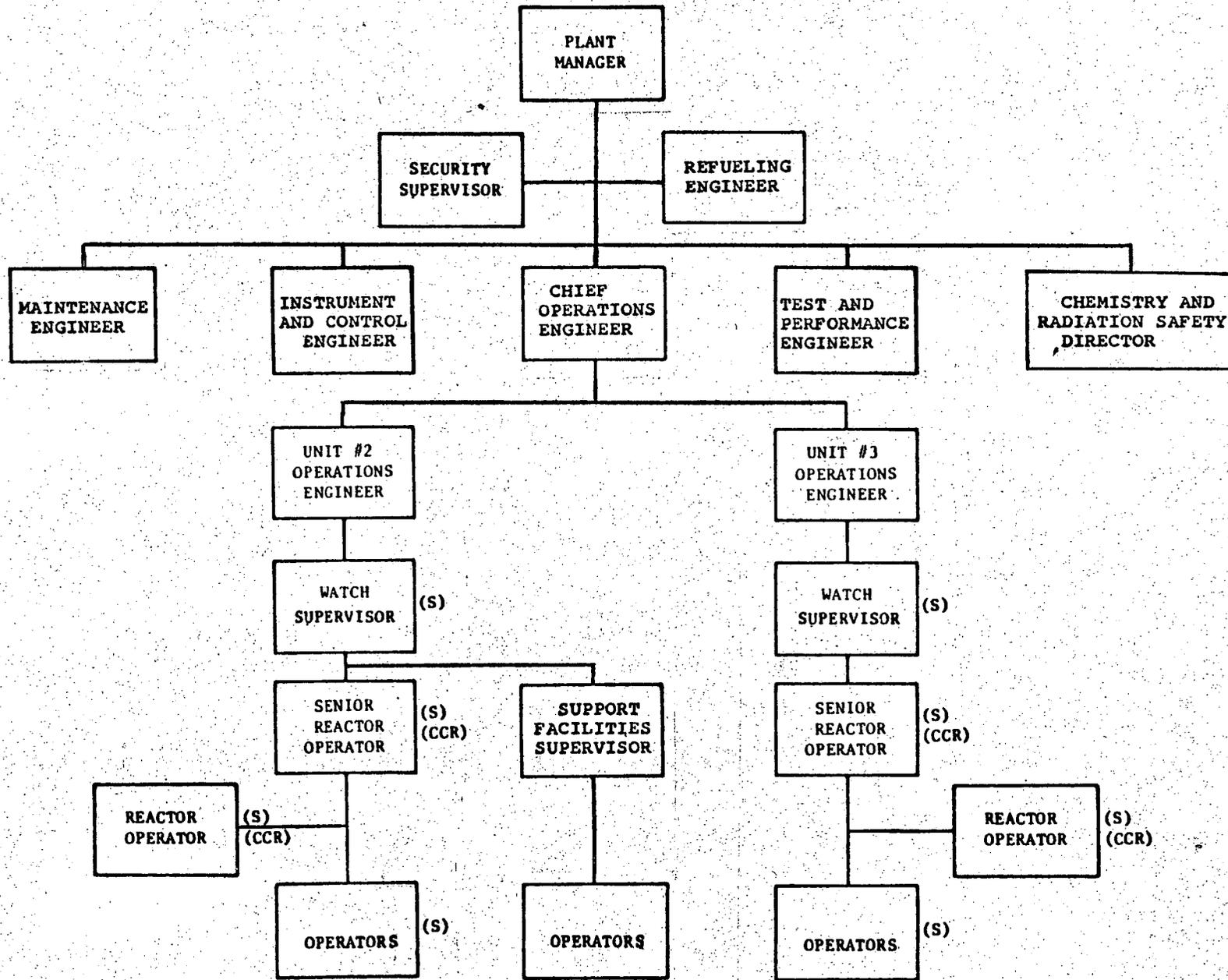


Figure 6.2-2 Facility Organization

(CCR) - Central Control Room Position
 (S) - Continuous Shift Coverage

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Director and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 STATION NUCLEAR SAFETY COMMITTEE (SNSC)

FUNCTION

6.5.1.1 The Station Nuclear Safety Committee shall function to advise the Plant Manager on all matters related to nuclear safety.

6.5.1.2 The Station Nuclear Safety Committee shall be composed as follows:

Chairman:	Plant Manager
Member:	Technical Engineering Director
Member:	Quality Assurance Engineer
Member:	Chief Operations Engineer
Member:	Security Supervisor
Member:	Test and Performance Engineer
Member:	Instrument and Control Engineer
Member:	Maintenance Engineer
Member:	Chemistry and Radiation Safety Director
Member:	Reactor Engineer

6.5.1.2.1 In the event the title of any job referred to in 6.5.1.2 is changed by the licensee, a report of such change shall be made within 30 days as provided in 6.9.2.e and the implementation of such change shall not be deemed a violation of the technical specifications.

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the SNSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SNSC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SNSC shall meet at least once per calendar month and as convened by the SNSC Chairman.

QUORUM

6.5.1.5 A quorum of the SNSC shall consist of the Chairman or Vice Chairman and five members including alternates.

RESPONSIBILITIES

- 6.5.1.6 The Station Nuclear Safety Committee shall be responsible for:
- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, and 2) any other proposed procedures or changes thereto as determined by the Plant Manager to affect nuclear safety.
 - b. Review of all proposed tests and experiments that affect nuclear safety.
 - c. Review of all proposed changes to the Technical Specifications.
 - d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
 - e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Manager, Nuclear Power Generation Department and to the Chairman of the Nuclear Facilities Safety Committee.
 - f. Review of facility operations to detect potential safety hazards.
 - g. Performance of special reviews and investigations and the issuance of reports thereon as requested by the Chairman of the Nuclear Facilities Safety Committee.
 - h. Review of the Plant Security Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.
 - i. Review of the Emergency Plan and implementing procedures and submission of recommended changes to the Chairman of the Nuclear Facilities Safety Committee.

AUTHORITY

- 6.5.1.7 The Station Nuclear Safety Committee shall:
- a. Recommend to the Plant Manager, in writing, approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
 - b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

AUTHORITY (Continued)

- c. Provide immediate written notification to the Chairman, Nuclear Facilities Safety Committee and the Manager, Nuclear Power Generation Department of disagreement between the recommendations of the SNSC and the actions contemplated by the Plant Manager. However, the course of action determined by the Plant Manager pursuant to 6.1.1 above shall be followed.

RECORDS

6.5.1.8 The Station Nuclear Safety Committee shall maintain written minutes of each meeting and copies shall be provided to, as a minimum, the Manager, Nuclear Power Generation Department and the Chairman, Nuclear Facilities Safety Committee.

6.5.2 NUCLEAR FACILITIES SAFETY COMMITTEE (NFSC)

FUNCTION

6.5.2.1 The Nuclear Facilities Safety Committee shall function to provide independent review and audit of designated activities in the areas of:

- a. reactor operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices
- i. environmental effects
- j. other appropriate fields associated with the unique characteristics of the nuclear power plant

RECORDS

6.5.2.10 Records of NFSC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NFSC meeting shall be prepared, approved and forwarded to the Senior Company Officer in charge of Power Supply within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Senior Company Officer in charge of Power Supply within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Company Officer in charge of Power Supply and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a Reportable Occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SNSC and submitted to the NFSC Chairman, and the Manager, Nuclear Power Generation Department.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Manager, Nuclear Power Generation Department and to the NFSC Chairman immediately.

- e. A written report shall be forwarded within 30 days to Director of Nuclear Reactor Regulation and to the Director of the Region 1 Office of Inspection and Enforcement following any change in the facility management and technical support organization or the facility organization, as shown on Figures 6.2-1 and 6.2-2, respectively, or any change in the job title of any member of the Station Nuclear Safety Committee as specified in 6.5.1.2.

6.10 RECORD RETENTION

- 6.10.1 The following records shall be retained for at least five years:
 - a. Records and logs of facility operation covering time interval at each power level.
 - b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
 - c. Reportable Occurrence Reports.
 - d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
 - e. Records of reactor tests and experiments.
 - f. Records of changes made to Operating Procedures.
 - g. Records of radioactive shipments.
 - h. Records of sealed source leak tests and results.
 - i. Records of annual physical inventory of all source material of record.

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ATTACHMENT B

APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

SAFETY EVALUATION

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2

Docket No. 50-247

November 8, 1976

SAFETY EVALUATION

The proposed changes to the Indian Point Unit No. 2 Technical Specifications, contained in Attachment A to this Application, would reflect pending organizational changes and add the requirement that Con Edison notify the NRC in writing within thirty (30) days of any change in the facility management and technical support organization or the facility organization, as shown on Figures 6.2-1 and 6.2-2, respectively, or any change in the job title of any member of the Station Nuclear Safety Committee as specified in 6.5.1.2. In addition the Station Nuclear Safety Committee membership would be modified in accordance with the proposed organizational changes and the Plant Manager would become chairman of the committee. The proposed organizational changes are consistent with the Standard Technical Specifications for Westinghouse Pressurized Water Reactors and the addition of the 30-day reporting requirement would assure that the NRC Regulatory Staff has the opportunity to review organizational changes to determine if they conform with regulatory requirements.

The proposed changes do not in any way alter the safety analyses performed for Indian Point Unit No. 2. The proposed changes have been reviewed by the Station Nuclear Safety Committee and the Con Edison Nuclear Facilities Safety Committee. Both committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types or increase in the amounts of effluents or any change in the authorized power level of the facility.