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1.8 Reportable Occurrence

A Reportable Occurrence shall be any of those conditions specified in Revision 4 of Regulatory Guide 1.16, "Reporting of Operating Information - Appendix "A" Technical Specifications" Sections C.2.a and C.2.b.

1.9 Quadrant Power Tilt

The quadrant power tilt is defined as the ratio of maximum to average of the upper excore detector currents or the lower excore detector currents whichever is greater. If one excore detector is out of service, the three in-service units are used in computing the average.

Basis:

Water inventory balances, monitoring equipment, radioactive tracing, boric acid crystalline deposits, and physical inspections can disclose reactor coolant leaks. Any leak of radioactive fluid, whether from the reactor coolant system primary boundary or not can be a serious problem with respect to in-plant radioactivity contamination and cleanup or it could develop into a still more serious problem; and therefore, first indications of such leakage will be followed up as soon as practicable.

Although some leak rates on the order of GPM may be tolerable from a dose point of view, especially if they are to closed systems, it must be recognized that leaks in the order of drops per minute through any of the walls of the primary system could be indicative of materials failure such as by stress corrosion cracking. If depressurization, isolation and/or other safety measures are not taken promptly, these small leaks could develop into much larger leaks, possibly into a gross pipe rupture. Therefore, the nature of the leak, as well as the magnitude of the leakage must be considered in the safety evaluation.

When the source of leakage has been identified, the situation can be evaluated to determine if operation can safely continue. This evaluation will be performed by the Plant Operating Staff and will be documented in writing and approved by either the Plant Manager or his designated alternate. Under these conditions, an allowable primary system leakage rate of 10 gpm has been established. This explained leakage rate of 10 gpm is also well within the capacity of one-charging pump and makeup would be available even under the loss of off-site power condition.

If leakage is to the containment, it may be identified by one or more of the following methods:

- a. The containment air particulate monitor is sensitive to low leak rates. The rates of reactor coolant leakage to which the instrument

3.2 CHEMICAL AND VOLUME CONTROL SYSTEM

Applicability

Applies to the operational status of the Chemical and Volume Control System.

Objective

To define those conditions of the Chemical and Volume Control System necessary to ensure safe reactor operation.

Specification

- A. When fuel is in the reactor there shall be at least one flow path to the core for boric acid injection.
- B. The reactor shall not be made critical unless the following Chemical and Volume Control System conditions are met.
 - 1. Two charging pumps shall be operable.
 - 2. Two boric acid transfer pumps shall be operable.
 - 3. The boric acid tanks together shall contain a minimum of 4400 gallons of 11 1/2% to 13% by weight (20,000 ppm to 22,500 ppm of boron) boric acid solution at a temperature of at least 145°F.
 - 4. System piping and valves shall be operable to the extent of establishing one flow path from the boric acid tanks and one flow path from the refueling water storage tank to the Reactor Coolant System.

OFFSITE

ONSITE

6-2

Change No.

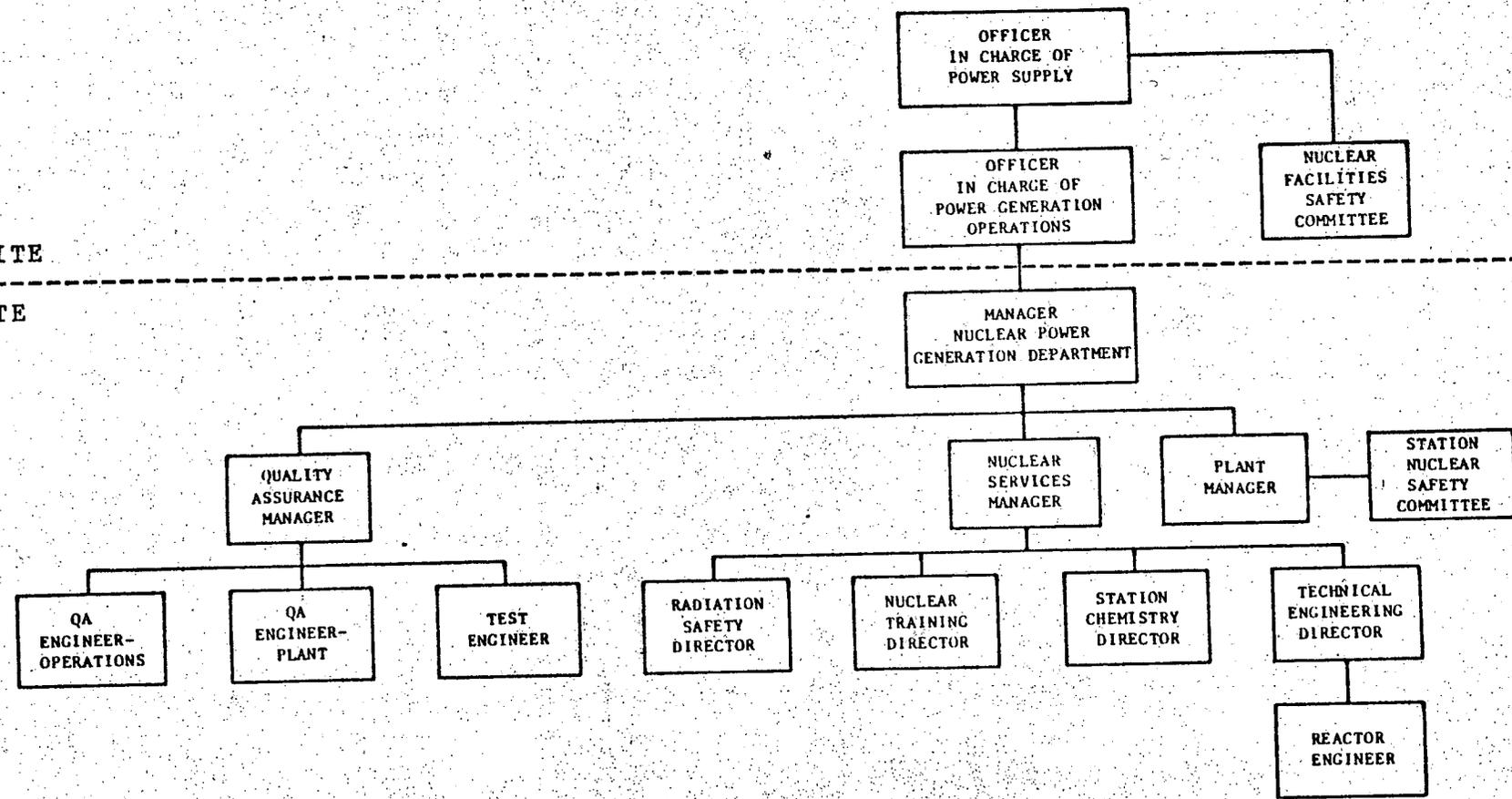


Figure 6.2-1 Facility Management and Technical Support Organization

REVIEW (Continued)

- g. Reportable Occurrences, as defined in Revision 4 of Regulatory Guide 1.16.
- h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the Station Nuclear Safety Committee.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the NFSC. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Facility Emergency Plan and implementing procedures at least once per two years.
- f. The Facility Security Plan and implementing procedures at least once per two years.
- g. Any other area of facility operation considered appropriate by the NFSC or the Senior Company Officer in charge of Power Supply.

AUTHORITY

6.5.2.9 The NFSC shall report to and advise the Senior Company Officer in charge of Power Supply on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NFSC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NFSC meeting shall be prepared, approved and forwarded to the Senior Company Officer in charge of Power Supply within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 e, f, g and h above, shall be prepared, approved and forwarded to the Senior Company Officer in charge of Power Supply within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Company Officer in charge of Power Supply and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a Reportable Occurrence:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each Reportable Occurrence Report submitted to the Commission shall be reviewed by the SNSC and submitted to the NFSC Chairman, the Plant Manager and the Manager, Nuclear Power Generation Department.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Manager, Nuclear Power Generation Department and to the NFSC Chairman immediately.

SAFETY LIMIT VIOLATION (Continued)

- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NFSC Chairman and the Manager, Nuclear Power Generation Department within 10 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of Regulatory Guide 1.33 (issued November, 1972) except as provided in 6.8.2 and 6.8.3 below.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and any changes to them shall be reviewed and approved for implementation in accordance with a written administrative control procedure approved by the Manager, Nuclear Power Generation Department, with the concurrence of the Station Nuclear Safety Committee and the Nuclear Facilities Safety Committee. The administrative control procedure required by this specification shall, as a minimum, require that:

- a. Each proposed procedure/procedure change involving safety related components and/or operation of same receives a pre-implementation review by the SNSC except in case of an emergency.
- b. Each proposed procedure/procedure change which renders or may render the Final Safety Analysis Report or subsequent safety analysis reports inaccurate and those which involve or may involve potential unreviewed safety questions are approved by the SNSC prior to implementation.
- c. The approval of the Nuclear Facilities Safety Committee shall be sought if, following its review, the Station Nuclear Safety Committee finds that the proposed procedure/procedure change either involves an unreviewed safety question or if it is in doubt as to whether or not an unreviewed safety question is involved.

6.8.3 A mechanism shall exist for making temporary changes and they shall only be made by approved management personnel in accordance with the requirements of ANSI 18.7-1972. The change shall be documented, and reviewed by the SNSC within 7 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE AND REPORTABLE OCCURRENCE REPORTS

6.9.1 Information to be reported to the Commission, in addition to the reports required by Title 10, Code of Federal Regulations, shall be in accordance with the Regulatory Position in Revision 4 of Regulatory Guide 1.16, "Reporting of Operating Information - Appendix "A" Technical Specifications".

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of Region 1, Office of Inspection and Enforcement within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Each containment integrated leak rate test shall be the subject of a summary technical report including results of the local leak rate tests since the last report. The report shall include analyses and interpretations of the results which demonstrate compliance in meeting the leak rate limits specified in the Technical Specifications.
- b. A report covering the X-Y xenon stability tests within three months upon completion of the tests.
- c. To provide the Commission with added verifications of the safety and reliability of the pre-pressurized Zircaloy-clad nuclear fuel, a limited program of non-destructive fuel inspections will be conducted. The program shall consist of a visual inspection (e.g., underwater TV, periscope, or other) of the two lead burnup assemblies in each region during the first, second, and third refueling shutdowns. Any condition observed by this inspection which would lead to unacceptable fuel performance may be the object of an expanded surveillance effort. If another domestic plant which contains pre-pressurized fuel of a similar design reaches fuel exposures equal to or greater than at Indian Point Unit, No. 2, and if a limited inspection program is or has been performed there, then the program may not have to be performed at Indian Point Unit No. 2. However, such action requires approval of the Nuclear Regulatory Commission. The results of these inspection will be reported to the Nuclear Regulatory Commission.
- d. A written report shall be forwarded within 30 days to the Division of Reactor Licensing and to the Director of the Region 1, Office of Inspection and Enforcement, in the event of:
 1. Discovery of the release of radioactive liquids, excluding tritium and dissolved noble gases exceeding 5 curies from the site during a consecutive 3 calendar month period.
 2. Discovery of the release of radioactive gases exceeding 50% of the limits specified in Specification 3.9.B.3.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. Reportable Occurrence reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all source material of record.

Regulatory Docket File

ATTACHMENT B

Received w/ Lic. Dated 1-7-76

APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

Description of Proposed Changes

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2
Docket No. 50-247

Facility Operating License No. DPR-26
January 6, 1976

Description of Proposed Changes

| <u>Page</u> | <u>Item</u> | <u>Existing Specification</u> | <u>Proposed Specification</u> | <u>Explanation of Change</u> |
|-------------|---------------|--|--|---|
| ii | 4.4 | "Recirculation..." | "Residual..." | Typographical error. |
| iii | 6.6 | "Abnormal Occurrence Action" | "Reportable Occurrence Action" | Per Reg. Guide 1.16, Rev. 4. |
| iii | 6.9 | "Routine and Abnormal Occurrence Reports" | "Routine and Reportable Occurrence Reports" | Per Reg. Guide 1.16, Rev. 4. |
| v | Second Figure | "2.2-2" | "2.1-2" | Typographical error. |
| 1-4 | 1.8 | <u>"Abnormal Occurrence"</u> | <u>"Reportable Occurrence"</u> | Per Reg. Guide 1.16, Rev. 4. |
| 1-4 | 1.8 | "An Abnormal Occurrence..." | "A Reportable Occurrence..." | Per Reg. Guide 1.16, Rev. 4. |
| 3.1-18 | Basis-3.1,F | "...General Superintendent..." | "...Plant Manager..." | Editorial Change to Reflect Current Organization. |
| 3.2-1 | 3.2.B.3 | "...25,500 ppm..." | "...22,500 ppm..." | Typographical error. |
| 6-2 | Figure 6.2-1 | Organizational Chart for Facility Management and Technical Support Organization. | Same chart but includes line separating onsite positions from offsite positions. | Editorial change to render the IP2 Technical Specifications consistent with the IP3 Technical Specifications. |
| 6-10 | 6.2.5.7,g | "ABNORMAL OCCURRENCES, as defined in Section 1.0 of these Technical Specifications." | "Reportable Occurrences, as defined in Revision 4 of Regulatory Guide 1.16." | Per Reg. Guide 1.16, Rev. 4. |

Description of Proposed Changes (Cont'd)

| <u>Page</u> | <u>Item</u> | <u>Existing Specification</u> | <u>Proposed Specification</u> | <u>Explanation of Change</u> |
|-------------|-------------|--|--|------------------------------|
| 6-11 | 6.6 | <u>"ABNORMAL OCCURRENCE ACTION"</u> | <u>"REPORTABLE OCCURRENCE ACTION"</u> | Per Reg. Guide 1.16, Rev. 4. |
| 6-11 | 6.6.1 | "...ABNORMAL OCCURRENCE:" | "...Reportable Occurrence:" | Per Reg. Guide 1.16, Rev. 4. |
| 6-11 | 6.6.1.b | "Each Abnormal Occurrence Report..." | "Each Reportable Occurrence Report..." | Per Reg. Guide 1.16, Rev. 4. |
| 6-12 | 6.8.1 | "...Appendix "A" of USAEC Regulatory Guide 1.33 except..." | "...Appendix "A" of Regulatory Guide 1.33 (issued November, 1972) except..." | Editorial change. |
| 6-13 | 6.9 | <u>"ROUTINE AND ABNORMAL OCCURRENCE REPORTS"</u> | <u>"ROUTINE AND REPORTABLE OCCURRENCE REPORTS"</u> | Per Reg. Guide 1.16, Rev. 4. |
| 6-13 | 6.9.1 | "...Revision 3..." | "...Revision 4..." | Per Reg. Guide 1.16, Rev. 4. |
| 6-14 | 6.10.1.c | "ABNORMAL OCCURRENCE REPORTS." | "Reportable Occurrence reports." | Per Reg. Guide 1.16, Rev. 4. |

ATTACHMENT C

APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

Safety Evaluation

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2
Docket No. 50-247

Facility Operating License No. DPR-26
January 6, 1976

Safety Evaluation

This Application for Amendment to Facility Operating License No. DPR-26 is limited to the effectuation of certain changes to the Appendix A Technical Specifications contained in that license. The proposed revisions would make certain editorial changes, including typographical corrections, and would make the specified reporting requirements subject to the current revision of the Regulatory Guide for reporting of operating information. Through the redefinition and clarification of terms contained in Revision 4 of Regulatory Guide 1.16, the proposed changes would bring Indian Point Unit No. 2 operating reporting requirements into line with requirements applied at other utilization facilities.

The proposed changes do not in any way alter the safety analyses performed for Indian Point Unit No. 2. The proposed changes have been reviewed by the Station Nuclear Safety Committee and by the Consolidated Edison Nuclear Facilities Safety Committee. Both committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types or increase in the amounts of effluents or any change in the authorized power level of the facility.

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

In the Matter of)
)
CONSOLIDATED EDISON COMPANY) Docket No. 50-247
OF NEW YORK, INC.)
(Indian Point Station,)
Unit No. 2))

CERTIFICATE OF SERVICE

I hereby certify that I have this 7th day of
January, 1976, served the foregoing document entitled
"Application for Amendment to Operating License" dated
January 6, 1976 by mailing copies thereof with first
class postage prepaid to the addressees listed below:

Hon. George V. Begany
Mayor, Village of Buchanan
Buchanan, New York 10511

Hendrick Hudson Free Library
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