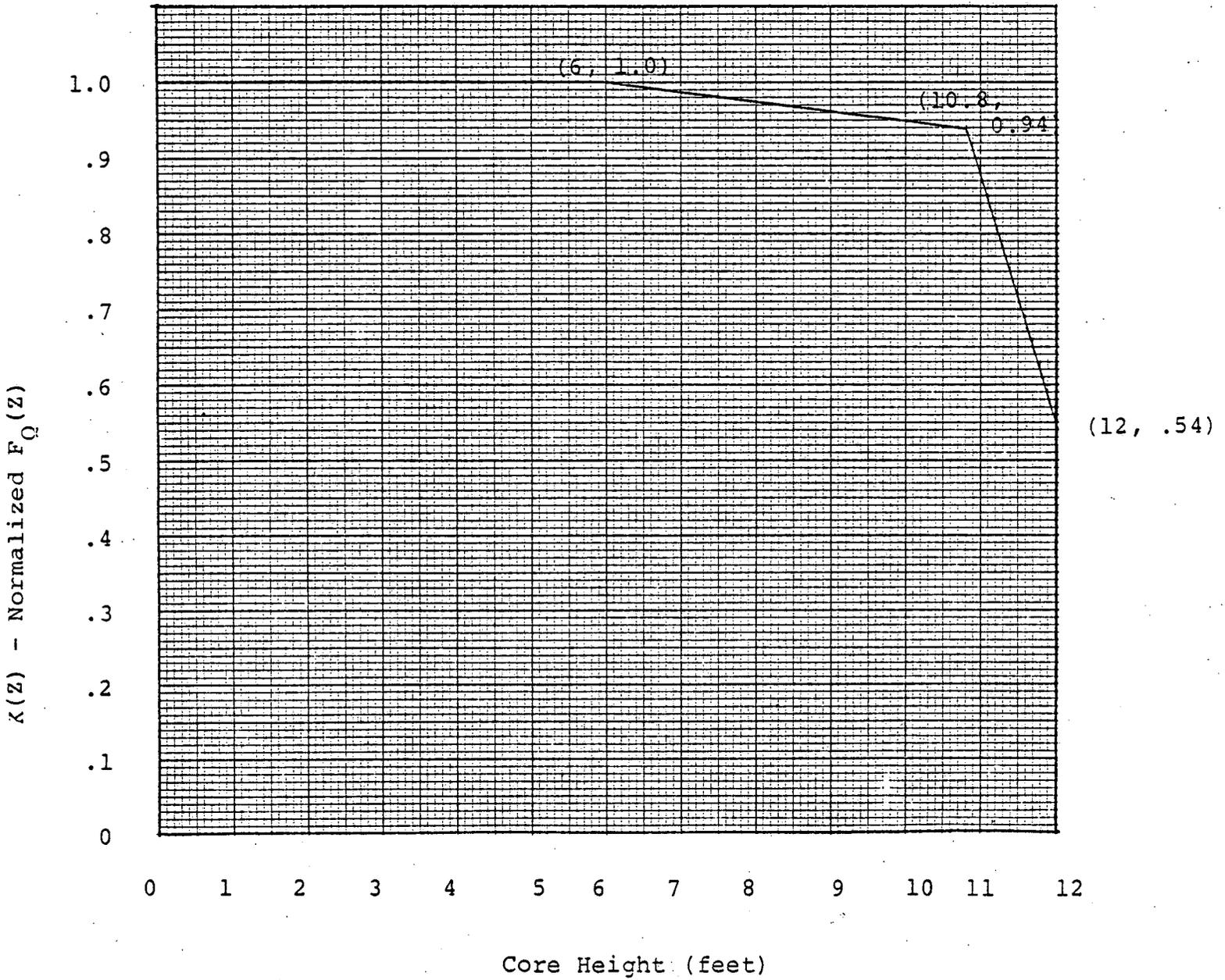


Figure 3.10-2

HOT CHANNEL FACTOR NORMALIZED OPERATING ENVELOPE

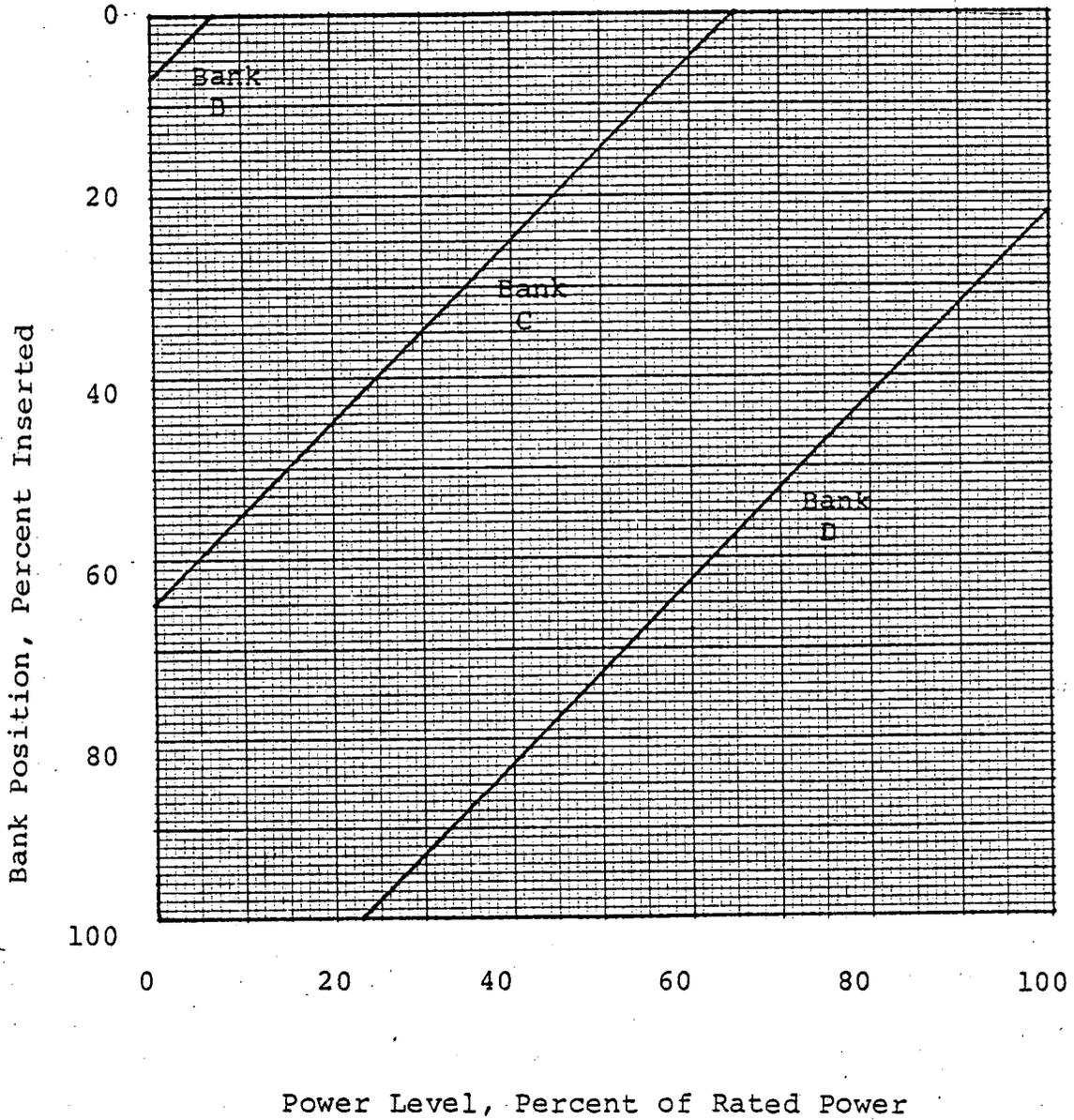


Amendment

8110310064 760209
PDR ADDCK 05000247
PDR

Figure 3.10-3

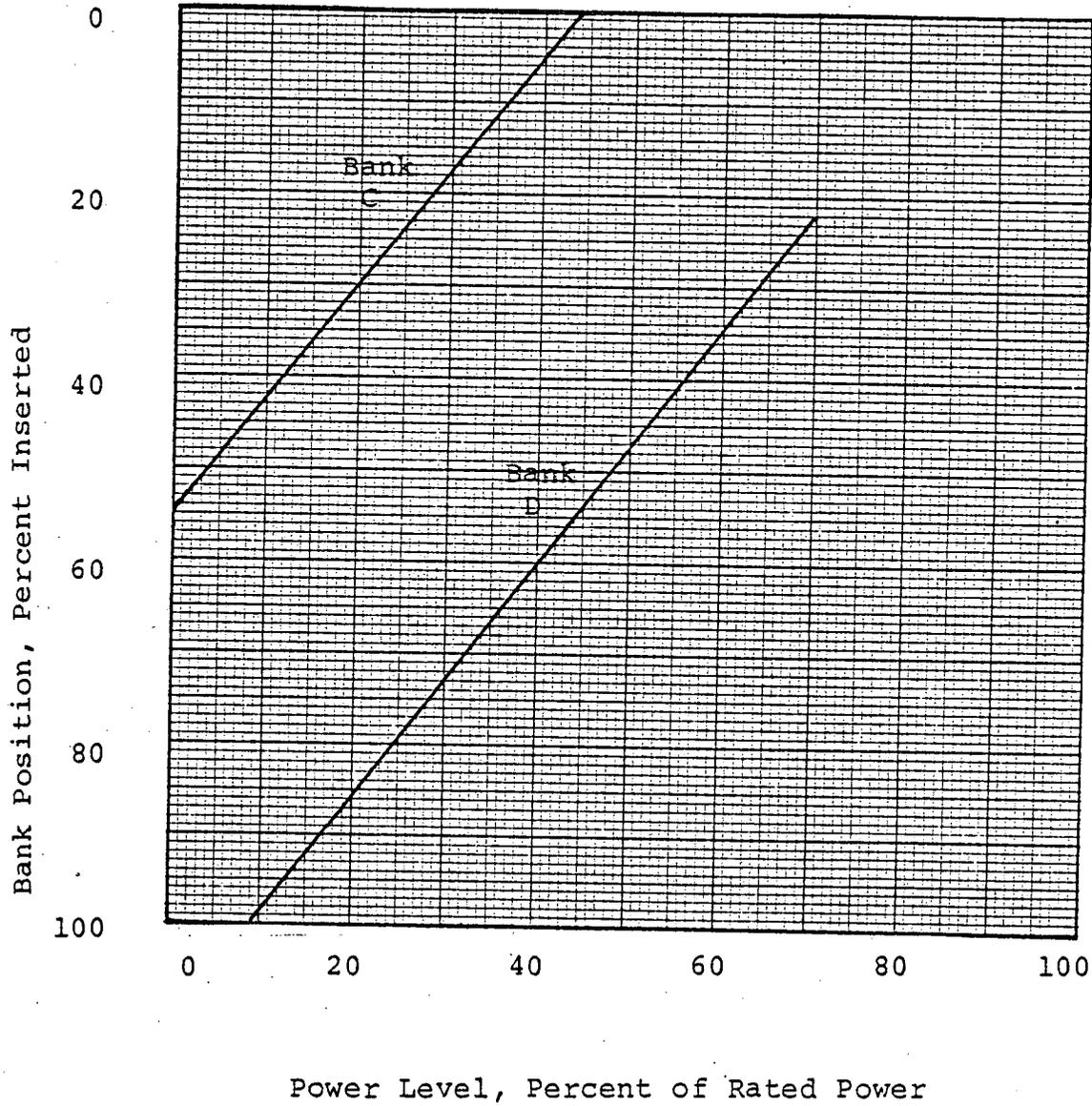
ROD BANK INSERTION LIMITS
(Four Loop Operation)
100 Step Overlay



Amendment

Figure 3.10-4

ROD BANK INSERTION LIMITS
(Three Loop Operation)
100 Step Overlap



Amendment

ATTACHMENT B

APPLICATION FOR AMENDMENT TO
OPERATING LICENSE

2-9-76

Regulatory Docket File

Safety Evaluation

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2
Docket No. 50-247

Facility Operating License, No. DPR-26

February 2, 1976

SAFETY EVALUATION

The proposed change to the three figures in Section 3.10 of the Technical Specifications and to the proposed Sections 3.10 of the Applications for Amendment to the License filed on September 6, 1974 and July 9, 1975 would make these figures consistent with the conclusions stated in the Reload Safety Evaluation for Indian Point Nuclear Plant Unit 2, Cycle 2. This Reload Safety Evaluation re-examines power capabilities and accident evaluations for Indian Point Unit No. 2. It is concluded in this Reload Safety Evaluation that the proposed Cycle 2 core reload and these proposed changes to the three figures in Section 3.10 of the Technical Specifications will not adversely affect the ability to safely operate at 100% of rated power during Cycle 2 and that the applicable design bases limits for the postulated incidents analyzed in the FSAR are not exceeded.

The proposed changes do not in any way alter the safety analyses performed for Indian Point Unit No. 2. The proposed changes have been reviewed by the Station Nuclear Safety Committee and by the Consolidated Edison Nuclear Facilities Safety Committee. Both committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types or increase in the amounts of effluents or any change in the authorized power level of the facility.