

ATTACHMENT A

APPLICATION FOR AMENDMENT  
TO OPERATING LICENSE

Technical Specification  
Page Revisions

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2  
Docket No. 50-247

Facility Operating License No. DPR-26  
May, 1977

8110310041 770505  
PDR ADOCK 05000247  
P PDR

<u>Section</u>	<u>Title</u>	<u>Page</u>
3.10	Control Rod and Power Distribution Limits	3.10-1
	Shutdown Reactivity	3.10-1
	Power Distribution Limits	3.10-1
	Quadrant Power Tilt Limits	3.10-4
	Rod Insertion Limits	3.10-5
	Rod Misalignment Limitations	3.10-6
	Inoperable Rod Position Indicator Channels	3.10-6
	Inoperable Rod Limitations	3.10-7
	Rod Drop Time	3.10-7
	Rod Position Monitor	3.10-7
	Quadrant Power Tilt Monitor	3.10-7
	Notification	3.10-8
3.11	Movable In-Core Instrumentation	3.11-1
3.12	Shock Suppressors (Snubbers)	3.12-1
4	Surveillance Requirements	4.1-1
4.1	Operational Safety Review	4.1-1
4.2	Inservice Inspection and Testing	4.2-1
4.3	Reactor Coolant System Integrity Testing	4.3-1
4.4	Containment Tests	4.4-1
	Integrated Leakage Rate Test - Pre-Operational	4.4-1
	Integrated Leakage Rate Test - Post-Operational	4.4-2
	Report of Test Results	4.4-4
	Continuous Leak Detection Testing via the Containment Penetration and Weld Channel Pressurization System	4.4-4
	Corrective Action	4.4-4
	Isolation Valve Tests	4.4-4
	Residual Heat Removal Systems	4.4-5
	Annual Inspection	4.4-6
	Containment Modification	4.4-6
4.5	Engineered Safety Features	4.5-1
	Safety Injection System	4.5-1
	Containment Spray System	4.5-2
	Hydrogen Recombiner System	4.5-2
	Component Tests	4.5-3
4.6	Emergency Power System Periodic Tests	4.6-1
	Diesel Generators	4.6-1
	Diesel Fuel Tanks	4.6-2
	Station Batteries	4.6-2
4.7	Main Steam Stop Valves	4.7-1
4.8	Auxiliary Feedwater System	4.8-1
4.9	Reactivity Anomalies	4.9-1
4.10	Environmental Monitoring Survey	4.10-1
4.11	Radioactive Materials	4.11-1
4.12	Shock Suppressors (Snubbers)	4.12-1
5	Design Features	5.1-1
5.1	Site	5.1-1
5.2	Containment	5.2-1
	Reactor Containment	5.2-1
	Penetrations	5.2-1
	Containment Systems	5.2-2
5.3	Reactor	5.3-1
	Reactor Core	5.3-1
	Reactor Coolant System	5.3-2
5.4	Fuel Storage	5.4-1

LIST OF TABLES

IP2 Reactor Vessel Core Region Material	3.1-1
Engineered Safety Features Initiation Instrument Setting Limits	3-1
Reactor Trip Instrumentation Limiting Operating Conditions	3-2
Instrumentation Operating Condition for Engineered Safety Features	3-3
Instrument Operating Conditions for Isolation Functions	3-4
<b>Safety Related Shock Suppressors (Snubbers)</b>	<b>3.12-1</b>
Minimum Frequencies for Checks, Calibrations and Tests of Instrument Channel	4.1-1
Frequencies for Sampling Tests	4.1-2
Frequencies for Equipment Tests	4.1-3
Minimum Shift Crew Composition	6.2-1
Protection Factors for Respirators	6.12-1

Amendment No.

#### 4.2. Inservice Inspection and Testing

Applicability Applies to the inservice inspection of Quality Groups\* A, B, and C components and the inservice testing of Quality Groups A, B and C pumps and valves.

Objective - To provide assurance of the continued integrity of those structures, components, and systems to which this specification is applicable.

#### Specification

4.2.1 Beginning November 1, 1977 the inservice inspection program of Quality Groups A, B, and C components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a (g), except where specific written relief has been granted by the NRC pursuant to 10CFR50, Section 50.55a (g) (6) (i). The program shall be updated every 40 months thereafter to reflect the provisions of the Code that have been both published by ASME and specifically incorporated by reference in Section 50.55a (b).

The applicable Code version shall be that in effect, as determined in accordance with the preceding sentence, six months prior to the commencement of each 40 month period. Changes in the applicable Code published by ASME and incorporated by reference in Section 50.55a (b) within such 6 month period may, in the discretion of the Licensee, be included in the program for the succeeding 40 month period.

\*Quality Group classification is in accordance with Revision 3 of Regulatory Guide 1.26.

4.2.2

Beginning November 1, 1977 the inservice testing program of Quality Groups A, B, and C pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10CFR50, Section 50.55a (g), except where specific written relief has been granted by the NRC pursuant to 10CFR50, Section 50.55a (g) (6) (i). The program shall be updated every 20 months thereafter to reflect the provisions of the Code that have been both published by ASME and specifically incorporated by reference in Section 50.55a (b).

The applicable Code version shall be that in effect, as determined in accordance with the preceding sentence, six months prior to the commencement of each 20 month period. Changes in the applicable Code published by ASME and incorporated by reference in Section 50.55a (b) within such 6 month period may, in the discretion of the Licensee, be included in the program for the succeeding 20 month period.

ATTACHMENT B

APPLICATION FOR AMENDMENT  
TO OPERATING LICENSE

Safety Evaluation

Consolidated Edison Company of New York, Inc.

Indian Point Unit No. 2  
Docket No. 50-247  
Facility Operating License No. DPR-26  
May, 1977

## Safety Evaluation

This application is submitted in accordance with the letters dated April 22, 1976 and November 17, 1976, from Mr. Robert W. Reid to Mr. William J. Cahill, Jr. The proposed changes to the Indian Point Unit No. 2 Technical Specifications, contained in Attachment A to the Application, incorporate the language suggested by the staff and comply with 10 CFR, Section 50.55a(g), as requested by the two referenced letters from Mr. Reid. The proposed changes would thereby assure that the inservice inspection program at Indian Point Unit No. 2 is performed in accordance with the ASME Section XI Code Edition that is currently "in effect", except where written relief has been granted by the NRC .

This amendment refers to Quality Groups A, B, and C, in lieu of Code Class 1, 2, and 3, because Code Class 1, 2, and 3, designations do not apply to Indian Point Unit No. 2 components, pumps and valves.

The proposed changes do not in any way alter the safety analyses performed for Indian Point Unit No. 2. The proposed changes have been reviewed by the Station Nuclear Safety Committee and the Con Edison Nuclear Facilities Safety Committee. Both committees concur that these changes do not represent a significant hazards consideration and will not cause any change in the types or increase in the amounts of effluents or any change in the authorized power level of the facility.

BEFORE THE UNITED STATES  
NUCLEAR REGULATORY COMMISSION

In the Matter of )  
 )  
CONSOLIDATED EDISON COMPANY ) Docket No. 50-247  
OF NEW YORK, INC. )  
(Indian Point Station, )  
Unit No. 2) )

CERTIFICATE OF SERVICE

I certify that I have, this 5th day of May, 1977,  
served the foregoing document entitled "Application for  
Amendment to Operating License" dated May 2, 1977 by  
mailing copies thereof, first class postage prepaid and  
properly addressed, to the following persons:

Hon. George V. Begany  
Mayor, Village of Buchanan  
188 Westchester Avenue  
Buchanan, New York 10511

Hendrick Hudson Free Library  
31 Albany Post Road  
Montrose, New York 10548

*Eugene R. Fidell*  
\_\_\_\_\_  
Eugene R. Fidell