

ALL COPY
AUG 2 1967

Docket No. 50-247

Mr. W. D. Crawford
Administrative Vice President
Consolidated Edison Company of New York, Inc.
4 Irving Place
New York, New York 10003

Dear Mr. Crawford:

On April 18, 1967, we received your sixth supplement to the Indian Point Nuclear Generating Station's Preliminary Safety Analysis Report, and subsequently, on July 18, 1967, we met with you and your representatives to discuss the contents of this report. As a result of this meeting and our review of the material presented in the report, we have found that the report is not responsive in many areas to the recommendations made by the Advisory Committee on Reactor Safeguards in their August 16, 1966 letter.

In particular, with reference to the Committee recommendation on the emergency core cooling system, the material presented in the report provides us with a more complete analysis of emergency core cooling than has heretofore been available for other pressurized water reactors. You will note, however, that the letter states that the AEC regulatory staff and the Committee should review the final design of the emergency core cooling system and the pertinent structural members within the pressure vessel. The report does not provide this design information. With reference to the water-cooled refractory-lined stainless steel tank, the design details and any other theoretical or experimental bases of the design that may be available should be provided.

With reference to the design and fabrication techniques and the in-service inspection possibilities of the primary system, the discussion in the report states, in effect, that code requirements will be met. The Committee's concern in this respect transcends normal code considerations and is an attempt to reduce still further the low probability of primary system rupture by encouraging design and fabrication techniques which are beyond normal code minimum requirements. Thus, you should present a discussion of the studies you have made which would demonstrate the adequacy of the design and fabrication techniques which you have used. In addition, you should present a description of the studies you have made concerning detection of incipient trouble in the primary system during reactor operation.

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With reference to studies of reactivity excursions, we find no reference to the fact that installation of solid burnable poison to reduce the positive moderate temperature coefficient is contemplated, nor do we find reference to partial length control rods for xenon oscillation control. We understand that the Committee is also interested in results of your investigations concerning the additional reactivity contribution of such effects as bowing of individual fuel rods or local boiling during a transient.

A report in the general framework outlined above with specific and complete replies to each of the recommendations made by the Advisory Committee on Reactor Safeguards is required. If you should need further clarification of any of these items, we are available to discuss it with you or your representatives.

Sincerely yours,

Original Signed by
Peter A. Morris

Peter A. Morris, Director
Division of Reactor Licensing

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