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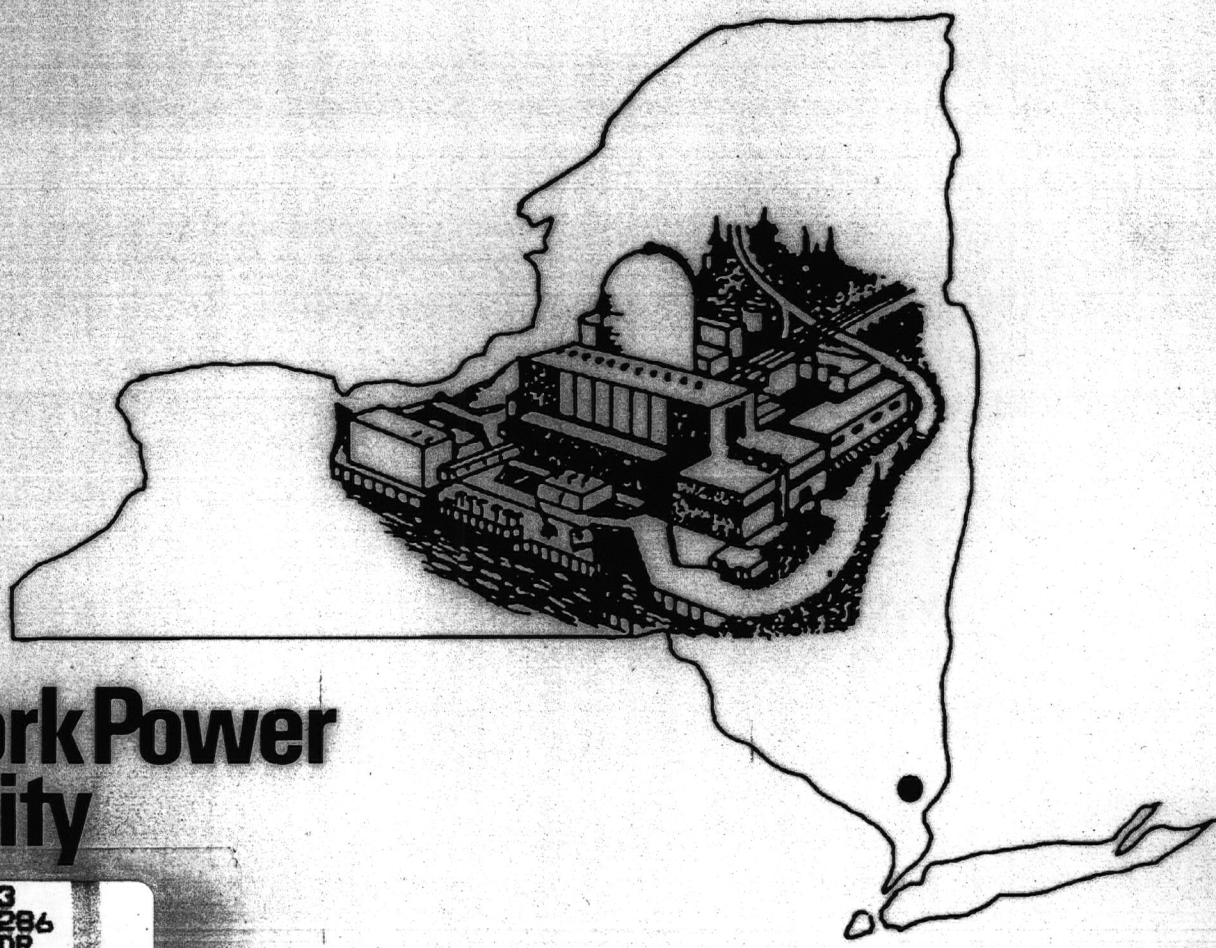
Regulatory Docket File

1991 PARTIAL EMERGENCY RESPONSE EXERCISE MANUAL

FOR THE

INDIAN POINT THREE NUCLEAR POWER PLANT MARCH 13, 1991



New York Power Authority

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NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC PARTIAL EXERCISE

MARCH 13, 1991

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SECTION 1

GROUND RULES AND SAFETY PRECAUTIONS

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511

914-736-8000

New York Power Authority

March 13, 1991

MEMORANDUM TO: ALL EXERCISE PARTICIPANTS

FROM: DAVE BELL
EMERGENCY PLANNING COORDINATOR

SUBJECT: EXERCISE GROUND RULES AND SAFETY PRECAUTIONS

In accordance with NUREG-0654 and the IP-3 Emergency Plan, a Partial Emergency Exercise is being conducted on March 13, 1991.

All exercise participants are required to observe the following Exercise Ground Rules for the duration of the exercise. If you have any questions regarding these Ground Rules, contact an Exercise Controller for clarification.

1. Take no actions that affect plant or non-exercise related operations.

2. Take immediate action(s) to restore safe operation if an unsafe condition exists. Ignore the exercise situation if actual safety becomes a concern.

3. Ensure that all communications indicate that "THIS IS A DRILL". Make a positive statement that you are making an exercise related message both at the beginning and end of all messages or conversations. If communication lines are kept open for extended periods, periodically repeat the statement.

4. Make all required notifications. These include: notification to site personnel, New York State and the Counties, NRC Headquarters and IP-3 Resident Inspector, ANI, INPO, and Con Edison. Be sure to indicate that "THIS IS A DRILL".

5. There will be one or more Observer/Controllers at each important location. These individuals will provide information that would normally be available at that location (e.g., reactor status in the Control Room, dose rate readings with field teams, etc.). Only selected parameters and readings will be provided. The selected information will be sufficient to make decisions in accordance with the IP-3 Emergency Plan. Use only the information provided. Do not improvise information.

6. In most cases, you are expected to perform all the tasks that would be required as a result of the simulated events, e.g. access information, utilize instrumentation, obtain any procedures, drawings, parts and tools needed to effect repair or "fixes". Controllers will provide clarification on actions which are to be simulated or are outside the scope of this exercise in order to keep the exercise progressing in accordance with the scenario.

7. Be sure the Observer/Controller is aware of your actions (e.g., do not dispatch a monitoring or repair team unless the Controller is aware of it; he may choose to send an Observer/ Controller with that team).

8. Offsite monitoring team Controller/Observers will inform teams to request information from them as they need it. They shall demonstrate use of the equipment before the drill data are provided to them.

9. Observer/Controllers will observe all aspects of the exercise in order to prepare an in-house evaluation of plans, procedures, training, and performance. NRC and other personnel will also be evaluating the performance of participants at each location.

10. "Dressing out" of some participants may be requested in accordance with the scenario and shall be consistent with actual radiological conditions.

11. Post accident samples will not be taken. However, teams may be chosen, briefed, and dispatched. Due to the condensed time frame of the scenario, the results of the sample analysis will be given to the team at the appropriate time.

12. Remaining strictly within the bounds of ALARA, no entries will be made into containment for drill purposes. Teams will don protective clothing and follow procedures up to the air lock but will simulate entry into containment.

13. If evacuation of onsite personnel is warranted, only a small group from the affected areas will be evacuated.

If during any part of the exercise you are having trouble accomplishing your required duties, confusion arises, or clarification is necessary, ask your Controller. Controller assistance or clarification does not necessarily imply failure on you part. Your Controller will know the limitations of information he can provide you and will assist you only to the extent necessary.

This exercise is conducted to evaluate our Emergency Plan. The Exercise is also a training vehicle for members of the IP-3 Emergency Response Organization to practice working together and with outside organizations. Please make note of any improvements in any area that you observe as a participant and submit them to the Observer/Controllers at the conclusion of the exercise.

Thank you for your participation and adherence to these rules.

SECTION 2

OBJECTIVES

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001

New York Power Authority

Joseph E. Russell
Resident Manager

November 26, 1990
IP3-90-013
IPO-90-108L

Docket No. 50-286
License No. DPR-64

Mr. William Lazarus, Chief
Emergency Preparedness Section
Facilities Radiological Safety & Safeguards Branch
Division of Radiation Safety & Safeguards
U.S. Nuclear Regulatory Commission - Region I
475 Allendale Road
King of Prussia, PA 19406

Dear Mr. Lazarus:

I am providing herewith exercise objectives for the Indian Point No. 3 Partial Scale Exercise currently scheduled for March 13, 1991. These objectives are submitted in accordance with FEMA Guidance Memorandum EX-3 as delineated in your letter dated July 14, 1989.

This Exercise will be conducted in accordance with 10CFR50 requirements. The New York Power Authority intends to test the site response to a simulated emergency condition at Indian Point 3. Active participation by New York State and the four surrounding counties is not anticipated.

Interaction between the NRC staff and NYPA concerning the scenario development for this exercise is anticipated and welcomed at this time.

Should you or your staff have any questions, please feel free to contact David D. Bell, Site Emergency Planning Coordinator at (914) 736-8403.

Respectfully,

Joseph E. Russell
Resident Manager
Indian Point 3 Nuclear Power Plant

JER/MAC/po

Enclosure

cc: Resident Inspector's Office - IP-#3
U.S. Nuclear Regulatory Commission
PO Box 337
Buchanan, New York 10511

SECTION 2

INDIAN POINT NO. 3 NUCLEAR POWER PLANT
1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

OBJECTIVES

A. PURPOSE

This document provides guidance for the conduct of the 1991 Partial Exercise to be held on March 13, 1991 at the Indian Point No. 3 Nuclear Power Plant (IP-3).

This exercise will demonstrate that various emergency preparedness functions outlined in the IP-3 Emergency Plan can be performed. It will include appropriate notifications to Westchester, Rockland, Putnam, and Orange Counties and New York State. However, since this is a partial exercise only, limited participation by these offsite agencies is expected. Active participation by Consolidated Edition (IP-2) is anticipated for offsite survey support. The exercise will be conducted during normal working hours.

At no time will the exercise be permitted to interfere with the safe operation of IP-3. To ensure this goal, plant management may suspend the exercise, or any part of it, for any period of time.

B. GENERAL OBJECTIVES

The primary objectives of the 1991 Partial Exercise are to evaluate the ability of the Emergency Response Organization to respond to a simulated accident at the IP-3 Site and to evaluate the adequacy of the IP-3 Emergency Plan and Implementing Procedures to provide appropriate direction to the emergency organization. To this end, the following general objectives will be demonstrated:

1. The IP-3 staff will activate the Station's Emergency Plan.

2. The IP-3 staff will classify simulated emergency conditions in accordance with the Site's Emergency Action Levels and Initiating Conditions.

3. The IP-3 staff will respond to a simulated emergency, make proper and timely notifications through the emergency classifications, and activate the emergency response facilities in an efficient and timely manner.

4. The IP-3 Facility Managers will demonstrate timely and effective decisions to mitigate the consequences of the event and clearly demonstrate "who is in charge".

5. The IP-3 staff will adequately, effectively, and properly utilize the IP-3 Emergency Response Facilities (ERFs) and their emergency response equipment (CR, OSC, TSC, EOF, JNC).

6. The IP-3 staff will formulate and take the actions necessary to protect station personnel based on plant parameters, inplant and onsite field surveys, and/or offsite field monitoring information.

7. The IP-3 staff will evaluate the source term and make dose projections based on plant parameters and field surveys.

8. The IP-3 staff will conduct and continuously maintain accountability of Site personnel.

9. The IP-3 staff will demonstrate shift staffing via roster only (except for Operations and Security).

10. The IP-3 staff will demonstrate communications among the licensee's emergency facilities including the use of back-up systems.

C. SPECIFIC OBJECTIVES

The following specific objectives are broken down by emergency response facility. These objectives were used to develop the exercise scenario and provide an aid to drill observers to evaluate the exercise response.

1. Control Room (CR)

   a. The CR staff will adequately turn over control of the event upon activation of the Emergency Operations Facility (EOF).

2. Operations Support Facility (OSC)

   a. The OSC Manager and OSC Team Leaders will organize, brief, and dispatch Repair and Corrective Action Teams as necessary.

   b. The IP-3 Health Physics staff will maintain proper radiological controls as necessary throughout the course of the event.

   c. The OSC staff will verbally communicate information between the OSC and Repair and Corrective Action Teams as necessary.

d. The OSC staff will assist in the evacuation of non-essential personnel from the site as necessary.

e. The OSC staff will coordinate and dispatch a Search and Rescue Team.

f. The use of back-up emergency power supplies will be demonstrated in the OSC.

3. Technical Support Center (TSC)

a. The TSC staff will perform engineering evaluations of required corrective actions.

b. The use of back-up emergency power supplies will be demonstrated in the TSC.

4. Emergency Operations Facility (EOF)

a. The EOF staff will adequately take control of the event and assume all notifications to State, Local, Federal levels of government, and offsite agencies in accordance with established procedures.

b. The EOF staff will coordinate with Westchester EOC County Officials the evacuations of non-essential site personnel as necessary.

5. Security

a. The IP-3 Security staff will control site access and site evacuation as directed by the Emergency Director.

6. Accountability

a. Accountability of personnel within the IP-3 protected area will be conducted within 30 minutes and maintained throughout the exercise.

7. Joint News Center (JNC)

a. The JNC will brief the media in an accurate, coordinated and timely manner through news releases and briefings.

8. Exercise and Drills

a. This Partial Exercise incorporates the following drills:

1. Communication Drill - The IP-3 staff will demonstrate the ability to notify and communicate with State and Local governments, and field assessment teams.

2. Health Physics Drill - The Health Physics staff will respond to and conduct analysis of simulated elevated samples and direct radiation measurements within the plant environment.

b. The exercise controllers will referee the exercise in accordance with the prescribed scenario time line.

c. The exercise controllers will provide scenario data and answer questions without prompting exercise players.

d. The exercise observers will adequately critique exercise performance.

D. INTENT OF THE IP-3 EXERCISE SCENARIO

This exercise is intended to demonstrate many, but not necessarily all, of the IP-3 capabilities to respond to a wide range of emergency conditions.

During development, the scenario is reviewed by a committee consisting of representatives from the IP-3 Emergency Planning Group, Training, Public Relations, Technical Services and the Headquarters Emergency Planning Group.

The scenario is designed to activate the IP-3 Emergency Plan through various action levels. Although this scenario accurately simulates operating events, it is not intended to assess the operators' diagnostic capabilities but rather provides sequences which ultimately demonstrate the operators' ability to respond to events which result in exercising emergency plans and procedures. Free play is encouraged and the controllers will interfere only if operator/player action prematurely terminates the exercise or excessively deviates from the exercise schedule.

Because of the limited participation expected by offsite agencies, the scope and time frame of the scenario has been

developed primarily to test the capability of the onsite response organization. The exercise is scheduled to start between 6 AM and 4 PM and will last approximately 7 hours.

E. CONCEPT OF OPERATIONS AND CONTROL OF THE EXERCISE

The Authority will supply official Controllers/Observers for locations where an emergency response is being demonstrated. Prior to the exercise, the Controllers and Observers will attend a briefing where they will be provided with locations, maps, time periods, technical information, exercise guidelines, and an evaluation checklist for their exercise assignments.

The exercise initiating events and information will be controlled by the Lead Controller at IP-3. The Lead Controller will have the responsibility to control and coordinate the time sequence of initiating events.

As initiating events are provided to the IP-3 Staff, they will determine the nature of the emergency and implement appropriate emergency response procedures. These procedures include a determination of the emergency classification and notification of the appropriate Federal, State, and Local authorities.

The simulated accident will continue to develop based on data and information provided to the Emergency Response Facility personnel by the Controllers. Certain inconsistencies (such as technical reasons for equipment failure) may be intentional. Such inconsistencies may be necessary due to the restrictions of simulating an accident that has never occurred, and the requirement to provide an exercise basis which tests the Site capabilities to the maximum extend feasible in a limited time. The Lead Controller shall have the authority to resolve or explain any inconsistencies or problems that may occur during the exercise. With the exception of such potential inconsistencies, the internal operation of the Site ERFs shall be consistent with their intended operation in a real emergency.

F. MAINTAINING EMERGENCY READINESS

Actions taken by the participants will not reduce plant or public safety. The potential for creating real radiological or other emergencies shall be specifically avoided. All messages about real events will be clearly identified. For example, a real message will be preceded with the words "This is NOT, repeat, NOT a drill message.

During the exercise, the ability to recognize real conditions will be maintained. The exercise will be terminated in the event a real emergency condition exists. The exercise scenario will not result in degradation of systems, equipment, or supplies, nor will it affect the detection, assessment, or emergency response capability of the plant.

SECTION 3

OPERATIONS AND CONTROL

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

SECTION 3

INDIAN POINT NO. 3 NUCLEAR POWER PLANT
1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

OPERATIONS AND CONTROL

I. INSTRUCTIONS FOR OBSERVERS AND CONTROLLERS

The 1991 Partial Exercise for the Indian Point No. 3 Nuclear Power Plant will be conducted in accordance with IP-3 Emergency Plan Implementing Procedure IP-1080 - "Conduct of Emergency Exercises and Drills". This procedure describes the types of exercises and drills mandated, planning criteria, responsibilities of Controllers and Observers, and follow-up procedures for critique, reporting, and corrective actions.

A. The following items will be simulated:

1. The taking of chemistry samples may be simulated. Teams will, however, be chosen, briefed, and dispatched. Because of the condensed time frame of the scenario, results of the samples will be provided to the teams consistent with the scenario time line.

2. "Dressing out" of some participants may be requested in accordance with the scenario and shall be consistent with actual radiological conditions.

3. Remaining strictly within the bounds of ALARA, no entries will be made into containment or other high radiation areas. (Entry into PAB cells will be at the discretion of the OSC Controller.)

4. Actions that affect plant or non-exercise related operations and equipment will be simulated. Repair teams may be required to actually accumulate necessary tools, repair parts, and reference materials to perform assigned tasks at the discretion of the Lead OSC Controller.

5. Evacuation of entire assembly areas will be simulated. If evacuation is ordered by the Emergency Director, this will be demonstrated by a small group from each affected area and will be evaluated by the Accountability Observer in that area in accordance with IP-1053.

6. Routine plant surveillances (i.e., RCS leakage) may or may not be simulated at the discretion of the Emergency Response Facility Controllers. In cases where such activities do not interfere with the condensed time frame of the scenario, it is expected that Controllers will direct that such activities be performed.

B. Emergency response activities may include the following:

1. Facility activation and staffing (Control Room, OSC, TSC, EOF, and JNC).

2. All notifications and communications.

3. Offsite field readings and sampling.

4. Dose assessment and protective action recommendations.

5. Radiological exposure control including control points, dosimetry issue, in-plant and offsite sampling, and Site access control.

6. Utilization of emergency response equipment including the MIDAS system computer, TSC computer system and monitors, and other specific equipment located in each facility as per the Emergency Plan.

7. Accountability will be demonstrated by all Site personnel with the exception of actual Watch stations.

C. In addition, all Controllers and Observers should note the following:

1. All communications leaving the Site must indicate that this is an exercise. The statement "THIS IS A DRILL" must precede and end all such transmissions. If Communicators fail to make this statement, Observers are to immediately correct this deficiency.

2. If evacuation of an assembly area has been called for and demonstrated, the Assembly Area will be considered evacuated. It will no longer be necessary for the Observer to provide any further information to the Area Accountability Officer. However, the Observer should then assist the Accountability Officer in maintaining order and controlling access and egress in the Assembly Area.

3. Controllers and Observers are required to be present at their assigned locations but are not considered to be "visible" to the Players. As such, they should not impede performance of the Players in any way. Observers for OSC Repair Teams should remain in the I&C Superintendent's Office until they are called upon by the OSC Lead Controller to accompany a team.

4. Prompting player actions is <u>not</u> allowed. You must only provide that information which has been provided to you for dissemination to the Players. Any changes or additions to the scenario must be coordinated through the Control Room and Facility Controllers. If it becomes necessary to issue contingency messages to keep the scenario on track, insure that all Controllers and Observers involved are aware of the change and the reasons for it.

5. In some instances, Observers will also function as Controllers (e.g., those in Assembly Areas, or those with survey teams or repair teams). You should initially tell Players how you will be providing information to them (i.e., if they look at their instruments, readings will be provided; if they report to investigate or repair something, a visual description will be provided, etc.).

6. Observers will complete and submit copies of their respective Observer Checklists to the Emergency Planning Coordinator. Copies of these checklists are attached (Attachment 1).

II. <u>OBSERVER/CONTROLLER ASSIGNMENTS</u> (See Attachment 2)

III. <u>EXERCISE SCHEDULE</u>

An Observer/Controller Orientation and Exercise Briefing will be conducted on Tuesday, March 12, 1991 at 1:00 PM at the IP-3 Training Center. The purpose of this meeting is to present the scenario that will be utilized, review the ground rules for the exercise, address any questions or concerns that the Observers or Controllers may have, and provide the necessary data, maps, field reports, etc. required for presentation to the participants.

The 1991 IP-3 Partial Exercise will be approximately 6 hours long and will be conducted during normal working hours on Wednesday, March 13, 1991.

A debriefing for all Observers and Controllers will be held at 8:00 AM on Thursday, March 14, 1991 at the IP-3 Training

Center. This will be followed by a formal exercise critique in the Admin. Conference Room at 11:00 AM. At this time, representatives from each Emergency Response Facility or function will present a short summary of the findings in his or her area of observation.

This presentation will include comments stating whether or not the objectives established for the exercise were met in their specific area of observation. Comments will also include correct as well as incorrect actions and should include recommendations for correcting inadequate or unsatisfactory performance and/or procedures.

The Observer/Controllers who will present comments are as follows:

Bryan Ray	-	Control Room
Karla Nelson	-	Technical Support Center
Bob Cullen	-	Operations Support Center
Mary Ann Chaubard	-	Emergency Operations Facility
Steve Horvath	-	Radiological Assessment
Mike Kyer	-	Accountability
Bill Heady	-	Security
Bonnie Bostian	-	Joint News Center

In accordance with IP-3 Emergency Plan Implementing Procedure IP-1080, an exercise report shall be prepared by the Lead Controller (or his designee) and submitted to the Superintendent of Power. This exercise report shall include the following as a minimum:

- The scenario;
- The assignment sheets;
- An overview of the experience and emergency preparedness;
- A listing of each noted shortcoming and associated recommended corrective action;
- Proposed Emergency Plan Corrective Action Report (EPCAR) for each noted deficiency.

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: CONTROL ROOM

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Did the Facility Manager demonstrate who was in charge?	___	___
2.	Did the Control Room classify the EAL correctly?	___	___
3.	Were notifications made within 15 min. of the classification?		
	a. RECS/SEMO RADIO	___	___
	b. ROSTER I	___	___
	c. NRC	___	___

4. What time were the classification and notification made?

EAL	Time	RECS	Plant Staff	NRC
NUE	____	____	_____	_____
ALERT	____	____	_____	_____
SAE	____	____	_____	_____
GE	____	____	_____	_____

5.	Were plant staff notified of the classification?	___	___
6.	Did the POM, SRO/SS, Control Room Communicator, HP Tech and Control Room Clerk follow their responsibilities as per Vol. II Activation Procedure?	___	___
7.	Was the Control Room set-up as per Vol. II Activation Procedure?	___	___
8.	Did Control Room experience any emergency plan equipment failures?	___	___

If so, what were the failures and how was the problem addressed:

_____

_____

_____

9.	Were the status boards updated in a timely manner?	___	___
10.	Was the ENS phone manned?	___	___
11.	Were log books maintained for plant status/activities/ communications?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(CONTROL ROOM CONTINUED)

		YES	NO
12.	Was the staffing adequate?	___	___
13.	Was shift turnover demonstrated?	___	___
14.	Was accountability conducted and maintained throughout the emergency?	___	___
15.	Were all objectives met?	___	___

If not explain:

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FEB 14 1991

OBSERVER EVALUATION FORM
(CONTROL ROOM CONTINUED)

Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: EOF

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Was the EOF activated in a timely manner?	___	___
	a. Time EOF was called for activation _____		
	b. Time EOF declared themselves operational _____		
2.	Was the transfer of "command and control" from Control Room to EOF adequate?	___	___
3.	Did Facility Manager demonstrate who was in charge?	___	___
4.	Was the EOF set-up as per Vol. II Activation Procedure?	___	___
5.	Was staff familiar with their responsibilities as per Vol. II Activation Procedure?	___	___
6.	Was staffing adequate?	___	___
7.	Did the EOF experience any emergency plan equipment failures:	___	___
	If so, what were the failures and how was the problem addressed:		
	_____		
	_____		
	_____		
	_____		
	_____		
8.	Did the EOF classify the EALs correctly?	___	___
9.	Were notifications made within 15 minutes of the classification?		
	a. RECS/SEMO RADIO	___	___
	b. NRC	___	___

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OBSERVER EVALUATION FORM
(EOF CONTINUED)

		YES	NO
10.	What time were the classification and notification made:		

EAL	Time	RECS	Plant Staff	NRC
NUE	____	____	_____	_____
ALERT	____	____	_____	_____
SAE	____	____	_____	_____
GE	____	____	_____	_____

11.	Were plant staff notified of the classification?	___	___
12.	Were periodic briefings held on plant status?	___	___
13.	Were status boards updated in a timely manner?	___	___
14.	Did ED or designee conference with County liaisons?	___	___
15.	Was site evacuation declared?	___	___
	(If so, did EOF staff coordinate with Westchester County officials upon site-evacuation?)	___	___
16.	Were log books maintained for plant status/activities communications?	___	___
17.	Was shift staffing/turnover demonstrated?	___	___

What problems were experienced:

_____

_____

_____

_____

18.	HPN phone manned?	___	___
19.	Was accountability conducted and maintained throughout the emergency?	___	___
20.	Were all objectives met?	___	___

If not explain. _____

_____

_____

_____

FEB 14 1991

OBSERVER EVALUATION FORM
(EOF CONTINUED)

21. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DA[illegible]: _____ LOCATION: DOSE ASSESSMENT

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Was habitability of the EOF conducted? a. At what time was it initiated _____ b. What was the frequency _____	___	___
2.	What time were off-site monitoring teams requested: _____		
3.	Was the need for KI assessed?	___	___
4.	Was the meteorological data updated every 15 min.?	___	___
5.	What time were dose projections started? _____		
	What source term was used? _____		
	Was a release in progress?	___	___
6.	Were new dose projections made with each significant change of meteorological or radiological conditions?	___	___
7.	Were protective action recommendations made:		
	a. Site personnel	___	___
	b. State/counties	___	___
8.	Was state/county protective action implementation consistent with NYPA recommendations?	___	___
9.	Were Reuter Stokes data obtained?	___	___
10.	Was a 24 forecast obtained?	___	___
11.	Was EOF ventilation in an internal recirculation mode?	___	___
12.	Was the Halon system placed on manual?	___	___
13.	Was control point established in the hall?	___	___
14.	Were Security Guards updated on plant status?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(DOSE ASSESSMENT CONTINUED)

	YES	NO
15. Were field data compared with dose projections?	___	___
Were they consistent?	___	___

If not, what decisions were made?

_____

_____

_____

_____

16. Were all objectives met?	___	___

If not, explain: _____

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FEB 14 1991

OBSERVER EVALUATION FORM
(DOSE ASSESSMENT CONTINUED)

1 Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: FIELD MONITORING

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Were NYPA staff familiar with Procedure IP-1011?	___	___
2.	What time were teams dispatched from:		
	a. OSC _____		
	b. Con Ed Environmental Lab _____		
3.	Did NYPA teams obtain equipment prior to leaving the protected area:		
	- vehicle keys	___	___
	- respirator	___	___
	- walkie talkie (ConEd frequency)	___	___
	- dosimetry	___	___
	- cellular phone handset	___	___
4.	Was the IP-1011 vehicle check list utilized during the vehicle check?	___	___
5.	Were problems were identified during the vehicle check?		
	If so, what were they?		
	_____		
	_____		
	_____		
6.	What time did the teams arrive at the EOF?		
	- NYPA site Perimeter _____		
	- NYPA off-site monitoring _____		
	- ConEd off-site monitoring _____		
7.	Was briefing to the teams adequate?	___	___
8.	Did the briefings include the following:		
	1-Plant conditions	___	___
	2-Meteorological conditions	___	___
	3-Types of readings to be obtained	___	___
	4-KI	___	___
	5-Exposure records	___	___
	6-Precautions with regard to field radiation levels	___	___
	7-Dosimetry recording	___	___
	8-General equipment precautions	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(FIELD MONITORING CONTINUED)

		YES	NO
9.	Was communication between the teams and EOF adequate?	___	___
10.	What mode of communication was used? a. radio _____ b. cellular phone _____ c. other (specify) _____		
11.	Was dosimetry checked and recorded?	___	___
12.	Were teams familiar with their sample locations?	___	___
13.	Were proper HP techniques utilized during sample collection and analysis?	___	___
14.	Was shift staffing/turnover demonstrated?	___	___

What problems were experienced:

_____

_____

_____

_____

15.	Were all objectives met?	___	___

If not, explain: _____

_____

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_____

_____

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_____

FEB 14 1991

OBSERVER EVALUATION FORM
(FIELD MONITORING CONTINUED)

16. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: OSC _____

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Was the OSC activated in a timely manner?	___	___
	a. Time OSC was called for activation _____		
	b. Time OSC declared themselves operational _____		
2.	Did Facility Manager demonstrate who is in charge?	___	___
3.	Was the OSC set-up as per Vol. II activation procedure?	___	___
4.	Was staff familiar with their responsibilities as per Vol. II Activation Procedure?	___	___
5.	Was staffing adequate?	___	___
6.	Did the OSC experience any emergency plan equipment failures?	___	___
	If so, what were the failures and how was the problem addressed:		
	_____		
	_____		
	_____		
	_____		
7.	Were plant staff notified of the classifications?	___	___
8.	Were periodic briefings held on plant status?	___	___
9.	Were status boards updated in a timely manner?	___	___
10.	Were Repair and Corrective Action Team status boards current with the teams activities?	___	___
11.	Were Repair & Corrective Action Team briefings adequate and timely?	___	___
12.	Were plant staff exposures obtained for R&CA team availability?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(OSC CONTINUED)

		YES	NO
13.	Were there any communication problems between the OSC & R&CA teams?	___	___
14.	Were overexposure authorizations necessary?	___	___
	If so, were actions consistent with procedures?	___	___
15.	Were log books maintained for plant status/activities/ communications?	___	___
16.	Was shift staffing/turnover demonstrated?	___	___

What problems were experienced:

_____

_____

_____

17.	Was HPN phone manned?	___	___
18.	Was accountability conducted and maintained throughout the emergency?	___	___
19.	Were plant announcements audible?	___	___
20.	Were all objectives met?	___	___

If not, explain: _____

_____

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FEB 14 1991

OBSERVER EVALUATION FORM
(OSC CONTINUED)

21. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: REPAIR & CORRECTIVE ACTION TEAMS

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Did the team consist of a minimum of two individuals?	___	___
2.	Was a briefing conducted?	___	___
	If so, did it include:		
	a. most direct route	___	___
	b. proper tools	___	___
	c. tasks understanding	___	___
	d. visual aids (maps)	___	___
	e. simulations	___	___
	f. radiation area dose rates	___	___
3.	Were the OSC Manager, POM and ED cognizant of all Repair and Corrective Action Team efforts?	___	___
4.	Did POM/SS approve work on safety related items?	___	___
5.	Was TSC direction obtained for engineering repair work?	___	___
6.	Was REA prepared?	___	___
7.	Was dosimetry, protective clothing etc. issued as per the REA?	___	___
8.	Were there any equipment failures?	___	___

If so, what were they and how was problem addressed?

_____

_____

_____

_____

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FEB 14 1991

OBSERVER EVALUATION FORM
(REPAIR & CORRECTIVE CONTINUED)

		YES	NO
9.	Was a debrief conducted?	___	___
10.	Were all objectives met?	___	___

If not, explain: _____

_____

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FEB 14 1991

OBSERVER EVALUATION FORM
(REPAIR & CORRECTIVE CONTINUED)

11 Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

I[illegible]: _____ LOCATION: HP TECHNICIANS

OBSERVER: _____ CONTROLLER: _____

WATCH H.P.

		YES	NO
1.	Did he/she account for all personnel in the Radiological Controlled Area?	___	___
2.	Did he/she report to Control Room to provide H.P. coverage?	___	___
	Time Watch H.P. arrived in the Control Room _____		
3.	Did he/she assist in the Control Room set-up?	___	___
4.	Was he/she familiar with the Emergency Plan procedures?	___	___

NON-WATCH CONTROL ROOM H.P.

1.	Was he/she familiar with their responsibilities as per Volume II Activation procedures?	___	___
2.	Did he/she use a log book to document events?	___	___
3.	Were ARM's/PRM's recorded?	___	___
	If yes, at what frequency?		
	_____		
	_____		
	_____		
4.	Were the TLD's/SRD's issued to Control Room personnel?	___	___
	If yes, were names and dosimetry numbers logged?	___	___
5.	Were contaminated checkpoints established at the Control Room access points?	___	___
6.	Were counting and air sampling equipment set-up?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(H.P. TECHNICIANS CONTINUED)

		YES	NO
7.	Were there any emergency plan equipment problems or failures?	___	___

If yes, what were they and how was the problem addressed?

_____

_____

_____

_____

8.	Were habitability surveys conducted?	___	___

If yes, at what frequency? _____

_____

_____

NON-WATCH OSC H.P.

1.	Was the TLD rack brought to the OSC?	___	___
2.	Did the H.P. technicians assist dosimetry bringing the TLD reader/cabinet to the OSC?	___	___
3.	Were they familiar with their responsibilities as per Volume II Activation procedures?	___	___
4.	Were they familiar with their OSC set-up as per Volume II Activation procedures?	___	___
5.	Were their any emergency equipment problems or failures?	___	___

If yes, what were they and how was the problem addressed?

_____

_____

_____

6.	Were control TLD's and dosimeters posted?	___	___
7.	Were habitability surveys conducted?	___	___

If yes, at what frequency? _____

FEB 14 1991

OBSERVER EVALUATION FORM
(H.P. TECHNICIANS CONTINUED)

		YES	NO
8.	Were ARM's/PRM's recorded?	___	___
	If yes, at what frequency? _____		
9.	Were contamination check points established?	___	___
10.	Were Repair and Corrective Action Team exposures logged?	___	___
11.	Were Repair and Corrective Action Team available doses calculated?	___	___

ALL H.P. POSITIONS

1.	Were all objectives met?	___	___

If not, explain: _____

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FEB 14 1991

OBSERVER EVALUATION FORM
(H.P. TECHNICIANS CONTINUED)

Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

I[illegible]E: _____ LOCATION: CHEMISTRY TECHNICIAN

OBSERVER: _____ CONTROLLER: _____

CONTROL ROOM WATCH CHEMIST

		YES	NO
1.	Did he/she report to Control Room upon implementing the Emergency Plan?	___	___
2.	What tasks were required by the Control Room for the Watch Chemist?		
	_____		
	_____		
	_____		
3.	Was the Chemist familiar with the procedures for the tasks?	___	___

OSC WATCH CHEMISTS

1.	Did they report to the OSC upon activation of the OSC?	___	___
2.	Was a Chemistry Tech. assigned as the OSC Area Communication System Communicator?	___	___
	If yes, was he/she familiar with the responsibility as per Volume II Activation procedure?	___	___
3.	What tasks were required by the Chemistry OSC Team Leader for the Chemists?		
	_____		
	_____		
	_____		
	_____		
	Were they familiar with the procedures for the tasks?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(CHEMISTRY TECHNICIANS CONTINUED)

		YES	NO
1.	Were all objectives met?	___	___

If not, explain:

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FEB 14 1991

OBSERVER EVALUATION FORM
(CHEMISTRY TECHNICIANS CONTINUED)

2 Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

D[illegible]: _____ LOCATION: TSC

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Was the TSC activated in a timely manner?	___	___
	a. Time TSC was called for activation _____		
	b. Time TSC declared themselves operational _____		
2.	Did Facility Manager demonstrate who is in charge?	___	___
3.	Was the TSC set-up as per Vol. II Activation Procedure?	___	___
4.	Was staff familiar with their responsibilities as per Vol. II Activation Procedure?	___	___
5.	Was staffing adequate?	___	___
6.	Did TSC experience any emergency plan equipment failures?	___	___

If so, what were the failures and how was the problem addressed:

_____

_____

_____

7.	Were plant staff notified of the classification?	___	___
8.	Were periodic briefings held on plant status?	___	___
9.	Were status boards updated in a timely manner?	___	___
10.	Were corrective action/solutions well thought out?	___	___

What documents were used for the solution?

_____

_____

_____

	Were contingency plans considered if solution didn't work?	___	___
11.	Did tech. staff reference EOP's and support the Control Room?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(TSC CONTINUED)

		YES	NO
12.	Were log books maintained for plant status/activities/ communications?	___	___
13.	Was shift staffing/turnover demonstrated?	___	___

What problems were experienced?

_____

_____

_____

_____

14.	Was accountability conducted and maintained throughout the emergency?	___	___
15.	Were all objectives met?	___	___

If not, explain:

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FEB 14 1991

OBSERVER EVALUATION FORM
(TSC CONTINUED)

16. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: ACCOUNTABILITY

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	If Alert was declared, did Accountability Officers man their Assembly Areas and perform their equipment checks?	___	___
2.	Did the Accountability Officers experience any emergency plan equipment failure?	___	___

If so, what were the failures and how was the problem addresssed?

_____

_____

_____

3.	Were Accountability Officers familiar with their responsibilities as per IP-1050?	___	___
4.	Did accountability begin at the Site Area Emergency?	___	___

If not, explain: _____

_____

_____

5.	Was accountability completed in 30 minutes?	___	___

If not, explain: _____

_____

_____

6. Time accountability declared: _____

Time protected area completed: _____

Time site accountability completed: _____

7.	Did LAO inform the ED and other assembly areas of accountability completion/status?	___	___

FEB 14 1991

OBSERVER EVALUATION FORM
(ACCOUNTABILITY CONTINUED)

		YES	NO
8.	Did LAO keep the assembly areas updated on plant conditions?	___	___
9.	Was LAO notified upon Site Evacuation?	___	___
10.	Were Accountability Officers familiar with radiation monitoring equipment?	___	___
11.	Were they familiar with the dose value to notify H.P. Team Leader?	___	___
12.	Were Assembly Area's evacuated?	___	___

Which one(s)? _____

_____

_____

13.	Was LAO notified upon that assembly area evacuation?	___	___
14.	Were all objectives met?	___	___

If not, explain: _____

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FEB 14 1991

OBSERVER EVALUATION FORM
(ACCOUNTABILITY CONTINUED)

15. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

DATE: _____ LOCATION: SECURITY

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Was staff familiar with their responsibilities as per Vol. II Activation Procedure?	___	___
2.	Was Security set-up as per Vol. II Activation Procedure?	___	___
3.	Were any equipment failures identified? If so, what were the failures and how was the problem addressed: _____	___	___
4.	Were staff aware of the EP Classifications and plant status?	___	___
5.	Was site access controlled? What time was it initiated? _____ What time was it terminated? _____	___	___
6.	Was site evacuation conducted?	___	___
	If yes, was it coordinated between the different organizations (security, accountability, OSC & EOF)?	___	___
7.	Were the accountability card readers activated?	___	___
8.	Were the Security guards in training notified upon TSC/OSC/EOF activation?	___	___
9.	Did Security assist in the accountability process?	___	___
10.	Were all objectives met?	___	___

If not, explain:

_____

_____

_____

_____

FEB 14 1991

OBSERVER EVALUATION FORM
(SECURITY CONTINUED)

1. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

E: _____ LOCATION: FIRE BRIGADE

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Were Fire Brigade members familiar with their procedures?	___	___
2.	Was the site fire alarm sounded?	___	___
3.	Did the control room announce over the PA system for the Fire Brigade to respond?	___	___
4.	Were all unnecessary personnel evacuated from the fire area?	___	___
5.	Was offsite assistance requested?	___	___
6.	Did radiological conditions exist at the fire scene?	___	___
	If so,		
	a. Were the radiological conditions assessed?	___	___
	b. Was dosimetry issued?	___	___
	c. Was an H.P. dispatched from the OSC?	___	___
	d. Did H.P. perform a radiological survey of the area including an air sample?	___	___
	e. Did H.P. check that the dosimetry was issued?	___	___
	f. If area dose rate >25 R/hr, was a briefing held?	___	___
	g. Was a debrief conducted?	___	___
7.	Were all objectives met?	___	___

If not, explain:

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FEB 14 1991

OBSERVER EVALUATION FORM
(FIRE BRIGADE CONTINUED)

8. Miscellaneous, Comments and Notes:

FEB 14 1991

OBSERVER EVALUATION FORM

D[illegible]: _____ LOCATION: JNC

OBSERVER: _____ CONTROLLER: _____

		YES	NO
1.	Was the JNC activated in a timely manner?	___	___
2.	How were JNC staff notified?		

_____

_____

_____

3.	When was the JNC activated? _____		
4.	Was the information flow from the plant, TSC, EOF to the JNC timely, accurate and complete?	___	___
5.	Did communications equipment operate efficiently?	___	___

If not, what were the problems and how were they addressed?

_____

_____

_____

6.	Were news releases timely, accurate and complete?	___	___

Number of news releases _____

Time of news releases _____

7.	Was technical review of news releases performed?	___	___
8.	Were posters accurate and updated in a timely manner?	___	___

Number of posters _____

Time of posters _____

9.	Were news briefings timely?	___	___

Number of news briefings _____

Time of briefings _____

FEB 14 1991

OBSERVER EVALUATION FORM
(JNC CONTINUED)

		YES	NO
10.	Did the briefings provide complete, accurate information?	___	___
11.	Were schematics used during briefings?	___	___
12.	Did the utility effectively share information with state and county PIO staff?	___	___
13.	Were support functions such as registration and security performed effectively?	___	___
14.	Were all the objectives met?	___	___

If not, explain: _____

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FEB 14 1991

OBSERVER EVALUATION FORM
(JNC CONTINUED)

Miscellaneous, Comments and Notes:

FEB 14 1991

Attachment 2

1991 EXERCISE ASSIGNMENT SHEET

MARCH 13, 1991

JOB FUNCTION	OBSERVER/CONTROLLER	PHONE #
Lead Controller	D. Bell	
Control Room Controller Plant Operations Manager Shift Supervisor Sr. Reactor Operator Reactor Operator	B. Ray	X 8274
CR Communicator(s) Accountability	M. McGough	
TSC Controller	K. Nelson	739-3099
TSC Manager TSC Mechanical TSC Electrical	S. Memon	
TSC Reactor	R. Kappes	
TSC Accountability		
OSC Controller	R. Cullen	X 8704 X 8715
OSC Manager OSC Team Leaders OSC Accountability OSC H.P. Control Point	A. Grosjean	
Watch H.P.	J. Darman	
Watch Chemist	J. Matwijiw	
EOF Controller	M. Chaubard	737-3629 X 8486
Emergency Director RATL Rad. Assessment Team: MIDAS Operator EOF Monitor Dose Assessment Rad. Comm./Survey Teams	S. Horvath	
Communicators Technical Advisor Public Relations P.R. Tech. Assistant EOF Accountability/Clerks	J. Kahabka	

Attachment 2

1991 EXERCISE ASSIGNMENT SHEET

MARCH 13, 1991

JOB FUNCTION	OBSERVER/CONTROLLER	PHONE#
Lead Accountability	M. Kyer	X 8497
Accountability Areas:		
Training	B. Martin	X 8667
Warehouse	D. Dicioccio	X 8158
2nd Floor Lunch Room	I. Catano/B. Taylor	X 8020
Construction	M. Campanaro	X 8699
Machine Shop	A. Radford	X 8625
Onsite Monitoring Team	M. Mozzor	
Offsite Monitoring Teams:		
(1)	A. Chan	
(2)	B. Hall	
Repair & Corrective Action Teams:		
(1)	P. Erickson	
(2)	D. Buyes	
(3)	S. Van Buren	
(4)	B. Swindell	
Security Command Post	E. Hojnacki	
Security Gates	M. Leonard	
Joint News Center	B. Bostian	X 8088 X 8089
Q.A.		

SECTION 4

SCENARIO OVERVIEW AND TIME LINE

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

SCENARIO OVERVIEW

MARCH 13, 1991

The Emergency Plan Exercise will commence at approximately 0830 hours with the plant at 100% power. At 0845 initiating conditions will exist for the declaration of an ALERT due to reactor coolant leakage exceeding 50 gallons per minute.

The Operations Support Center and Technical Support Center will be activated with the primary interfacility communication system (Five-Party Phone) failing to operate.

At 0915, a leak will occur in Component Cooling Water Heat Exchanger 31 from the Service Water System.

Initiating conditions will exist at 1015 for the declaration of a SITE AREA EMERGENCY (Reactor coolant leakage exceeding the capacity of two Charging Pumps). The Engineered Safeguard System will be activated with 35 Containment Fan Cooler Unit failing to start.

At approximately 1100, site accountability will be complete with a missing person identified.

The Vapor Containment Area Radiation Monitors (R-2 and R-7) will be slightly elevated due to the reactor coolant leakage. No fuel damage will occur.

At 1140, the IP-3 Plant Ventilation Radiation Monitor (R-27) will fail to operate.

At 1240, the Technical Support Center will experience a power failure. Start-up and transfer of loads to the TSC diesel will be simulated.

At 1345, the Control Room operators will be initiating RHR.

The drill terminates at 1415.

IP-3 1991 EMERGENCY PLAN EXERCISE
MARCH 13, 1991
INITIATING CUES/EXPECTED RESPONSES

0830 Initiating Cue

A message with plant data will be provided to the drill players in the IP-3 Control Room indicating current plant conditions for the day.

If the plant is not operating at 100% power, drill initial conditions will be provided reflecting 100% power operation.

Expected Response: None

0845 Initiating Cue

A message with plant data will reflect a reactor coolant leak from pressurizer safety valve 464.

Expected Response:

The operators will assess the source of the leak, magnitude of the leak, and capability to stop the leak. They should realize that the leak is not capable of being isolated and the magnitude is greater than 50 gallons per minute (the data is based on a 75 gallon per minute leak).

The plant status log data sheets reflect the operators initiating a plant shutdown (with hot shutdown being achieved in one hour). A plant cooldown at the rate of 40°F/hr. is initiated at 0945.

An ALERT Emergency should be declared due to the reactor coolant leakage exceeding fifty gallons per minute. Notification to outside agencies should follow with updates being provided every thirty minutes.

The Operations Support and Technical Support Centers should be activated.

0915 Initiating Cue

A message will be provided to the Control Room indicating inleakage into the Component Cooling Water System.

Expected Response:

The operators will assess the magnitude and source of the inleakage. As the Component Cooling Water (CCW) System Radiation Monitors (R-17a and 17b) do not indicate an increase in activity, they may conclude service water is the source of inleakage.

Chemistry Technician from the OSC may be sent to sample the CCW system to verify Service Water is the source of inleakage.

Operators may be dispatched to provide isolation of the respective heat exchangers in order to identify which one is the source of the leak.

Operators may be dispatched to align Waste Holdup Tanks 32 and 33 to accept fluid from 31 Waste Holdup Tank (which is accommodating the "overflow" from the CCW system).

TSC personnel may be calculating the ability of the CCW system to provide a sufficient heat sink (with one heat exchanger isolated).

TSC personnel may be determining various methods of isolating the CCW system leaks including the affect of reduced Service Water pressure, allowing for operation of the system with inleakage (and the capacity of the waste system) or the potential of overpressuring the CCW system.

0930 Initiating Cue

A message will be provided to each Emergency Response Center's five party communicator that the primary interfacility communication system (5-party phone) has failed.

Expected Response:

The Center's communicator should identify the problem to the Facility Manager and establish communication using a back-up system. It is anticipated that the four-party phone will be used.

An Instrument and Control team should be provided to determine the need for AT&T to repair the line.

1015 Initiating Cue

A message will be provided to Control Room players indicating that an automatic actuation of the plant's Engineered Safeguards System has occurred due to low pressurizer pressure.

Upon the operators verifying the automatic actions, Containment Fan Cooler Unit 35 fails to start.

Expected Response:

The operators should assess the cause of the Engineered Safeguards System (ESS) actuation and realize that the leak rate has increased. The increase is due to a waterhammer effect on the valve internals from the continuous discharge of the steam/water mixture. The data reflect a leak rate of 400

gallons per minute. The plant's Emergency Operating Procedures should be implemented with the operators verifying all the E.S.S. automatic actions and an initiation of a plant "cooldown". The data reflect a reactor coolant system cooldown rate of 40°F per hour commencing at 0945.

A SITE AREA EMERGENCY should be declared based upon known loss of coolant exceeding the capacity of two operable charging pumps. Notification of the upgrade should follow.

Site accountability should be initiated and completed within thirty minutes.

The Emergency Operating Facility should be activated with the transfer of command (from the control room) following.

A message will be provided to the operator that Containment Fan Cooler Unit 35 has failed to start. The operator will respond by describing efforts to manually start the unit using the Control Room switch. This action will not start the unit. A Repair/Corrective Action Team may be dispatched. The unit being out of service has no impact on the scenario.

1100 (approximately) Initiating Cue

Site accountability activities should identify a missing person.

Expected Response

A Search and Rescue Team should be dispatched to locate the missing person.

1100-1400 Initiating Cue

Data/messages have reflected slightly elevated radiation levels in the containment building due to the leakage of reactor coolant (no fuel damage exists).

Expected Response:

EOF personnel should:

- Calculate dose projections
- Discuss plant prognosis
- Discuss recommendations for protective actions

No recommendations for protective actions are anticipated.

SECTION 4

SCENARIO OVERVIEW AND TIME LINE

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

SCENARIO OVERVIEW

MARCH 13, 1991

The Emergency Plan Exercise will commence at approximately 0830 hours with the plant at 100% power. At 0845 initiating conditions will exist for the declaration of an ALERT due to reactor coolant leakage exceeding 50 gallons per minute.

The Operations Support Center and Technical Support Center will be activated with the primary interfacility communication system (Five-Party Phone) failing to operate.

At 0915, a leak will occur in Component Cooling Water Heat Exchanger 31 from the Service Water System.

Initiating conditions will exist at 1015 for the declaration of a SITE AREA EMERGENCY (Reactor coolant leakage exceeding the capacity of two Charging Pumps). The Engineered Safeguard System will be activated with 35 Containment Fan Cooler Unit failing to start.

At approximately 1100, site accountability will be complete with a missing person identified.

The Vapor Containment Area Radiation Monitors (R-2 and R-7) will be slightly elevated due to the reactor coolant leakage. No fuel damage will occur.

At 1140, the IP-3 Plant Ventilation Radiation Monitor (R-27) will fail to operate.

At 1240, the Technical Support Center will experience a power failure. Start-up and transfer of loads to the TSC diesel will be simulated.

At 1345, the Control Room operators will be initiating RHR.

The drill terminates at 1415.

IP-3 1991 EMERGENCY PLAN EXERCISE
MARCH 13, 1991
INITIATING CUES/EXPECTED RESPONSES

0830 Initiating Cue

A message with plant data will be provided to the drill players in the IP-3 Control Room indicating current plant conditions for the day.

If the plant is not operating at 100% power, drill initial conditions will be provided reflecting 100% power operation.

Expected Response: None

0845 Initiating Cue

A message with plant data will reflect a reactor coolant leak from pressurizer safety valve 464.

Expected Response:

The operators will assess the source of the leak, magnitude of the leak, and capability to stop the leak. They should realize that the leak is not capable of being isolated and the magnitude is greater than 50 gallons per minute (the data is based on a 75 gallon per minute leak).

The plant status log data sheets reflect the operators initiating a plant shutdown (with hot shutdown being achieved in one hour). A plant cooldown at the rate of 40°F/hr. is initiated at 0945.

An ALERT Emergency should be declared due to the reactor coolant leakage exceeding fifty gallons per minute. Notification to outside agencies should follow with updates being provided every thirty minutes.

The Operations Support and Technical Support Centers should be activated.

0915 Initiating Cue

A message will be provided to the Control Room indicating inleakage into the Component Cooling Water System.

Expected Response:

The operators will assess the magnitude and source of the inleakage. As the Component Cooling Water (CCW) System Radiation Monitors (R-17a and 17b) do not indicate an increase in activity, they may conclude service water is the source of inleakage.

Chemistry Technician from the OSC may be sent to sample the CCW system to verify Service Water is the source of inleakage.

Operators may be dispatched to provide isolation of the respective heat exchangers in order to identify which one is the source of the leak.

Operators may be dispatched to align Waste Holdup Tanks 32 and 33 to accept fluid from 31 Waste Holdup Tank (which is accommodating the "overflow" from the CCW system).

TSC personnel may be calculating the ability of the CCW system to provide a sufficient heat sink (with one heat exchanger isolated).

TSC personnel may be determining various methods of isolating the CCW system leaks including the affect of reduced Service Water pressure, allowing for operation of the system with inleakage (and the capacity of the waste system) or the potential of overpressuring the CCW system.

0930 Initiating Cue

A message will be provided to each Emergency Response Center's five party communicator that the primary interfacility communication system (5-party phone) has failed.

Expected Response:

The Center's communicator should identify the problem to the Facility Manager and establish communication using a back-up system. It is anticipated that the four-party phone will be used.

An Instrument and Control team should be provided to determine the need for AT&T to repair the line.

1015 Initiating Cue

A message will be provided to Control Room players indicating that an automatic actuation of the plant's Engineered Safeguards System has occurred due to low pressurizer pressure.

Upon the operators verifying the automatic actions, Containment Fan Cooler Unit 35 fails to start.

Expected Response:

The operators should assess the cause of the Engineered Safeguards System (ESS) actuation and realize that the leak rate has increased. The increase is due to a waterhammer effect on the valve internals from the continuous discharge of the steam/water mixture. The data reflect a leak rate of 400

gallons per minute. The plant's Emergency Operating Procedures should be implemented with the operators verifying all the E.S.S. automatic actions and an initiation of a plant "cooldown". The data reflect a reactor coolant system cooldown rate of 40°F per hour commencing at 0945.

A SITE AREA EMERGENCY should be declared based upon known loss of coolant exceeding the capacity of two operable charging pumps. Notification of the upgrade should follow.

Site accountability should be initiated and completed within thirty minutes.

The Emergency Operating Facility should be activated with the transfer of command (from the control room) following.

A message will be provided to the operator that Containment Fan Cooler Unit 35 has failed to start. The operator will respond by describing efforts to manually start the unit using the Control Room switch. This action will not start the unit. A Repair/Corrective Action Team may be dispatched. The unit being out of service has no impact on the scenario.

1100 (approximately) Initiating Cue

Site accountability activities should identify a missing person.

Expected Response

A Search and Rescue Team should be dispatched to locate the missing person.

1100-1400 Initiating Cue

Data/messages have reflected slightly elevated radiation levels in the containment building due to the leakage of reactor coolant (no fuel damage exists).

Expected Response:

EOF personnel should:

- Calculate dose projections
- Discuss plant prognosis
- Discuss recommendations for protective actions

No recommendations for protective actions are anticipated.

1140 Initiating Cue

Data will reflect that the Plant Vent Radiation Monitor (R-27) has failed to operate.

Expected Response:

A Repair and Corrective Action Team will be dispatched to repair the monitor. The unit being out of service has no impact on the scenario.

Dose assessment personnel will use back-up radiation monitors.

1240 Initiating Cue

A message will be given to TSC and OSC personnel that all power has been lost in the TSC/OSC except that powered by UPS. Power will be returned after approximately 10 sec. at which time the diesel will pick up the load.

Expected Response:

TSC/OSC personnel should assess that the normal power supply to the Centers has failed and the back-up supply (TSC Diesel Generator) has automatically started to provide power. An Operations Team should be dispatched to the TSC diesel to ensure operation. A Repair and Correction Action Team should be sent to determine the loss of power and provide corrective action.

1345 Initiating Cue

Data will reflect the reactor coolant system is <350°F and RHR can be initiated.

Expected Response:

RHR should be placed in service and cooldown continued to "cold shutdown".

1415 Drill Terminated

SCENARIO TIME LINE

MARCH 13, 1991

0830	Drill Commences - Current plant conditions for that day.
0845	Indication of a 75 gpm RCS leak
0900	ALERT declared (primary coolant leak rate greater than 50 gpm) - OSC/TSC activation - Notifications
0915	CCW Heat Exchanger leak
0930	Five (5) Party Phone failure - Repair/Corrective Action Team dispatched
0945	Plant at Hot Shutdown
1015	Indication of an increase in RCS leakage to 400 gpm - ESS activation (Containment Fan Cooler Unit 35 fails to start)
1030	SITE AREA EMERGENCY declared (known loss of coolant accident that exceeds the capacity of two operable charging pumps). - Accountability commences - EOF activated
1100	- Search/Rescue Team dispatched for missing person - Dose projection calculations - Protective Action Recommendations discussed - Containment atmosphere sample team dispatched
1140	- Indication of R-27 Plant Vent Rad Monitor failure. - Repair/Corrective Action Team dispatched - Plant Vent Sample Team dispatched.
1240	- Indication of TSC power failure - Repair/Corrective Active Team dispatched - TSC Diesel operation team dispatched
1345	- Initiating RHR
1415	- Drill terminated

SECTION 5

PLANT DATA

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 0830

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 1/1A

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Actual plant conditions will be used to commence the exercise.	None	N/A

If the plant is not at 100% power, Message 1A will commence the drill as follows:

CCR	The plant is at 100% power. It has been operating at 100% since returning from a refueling outage 60 days ago. Plant parameters are listed on plant status logs #1A.	None	N/A

NOTE: Waste Holdup Tank 31 level indicator on the Waste Disposal Panel is out of service.

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 0830

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 1

Actual plant conditions exist.

Waste Holdup Tank 31 level indication on the Waste Disposal Panel is out of service.

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL-

DATE: March 13, 1991 TIME: 0830

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 1A

The plant is operating at 100% power. It has been operating at 100% power since returning from a refueling outage sixty days ago.

No major equipment is out of service.

Plant parameters are as listed on Forms 31a, b and c.

Waste Holdup Tank 31 level indication on the Waste Disposal Panel is out of service.

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

Note:
To be provided with Message #1A ONLY

EP-FORM #31a | IP-3 PLANT STATUS LOG | DATE: MAR 13 1991

PARAMETER	TIME 0830	TIME	TIME	TIME	TIME
RCS Incore T/C Average (°F)	594				
RCS Incore T/C Highest (°F)	595				
RCS Average Temperature (°F)	566				
RCS Pressure (PSIG)	2235				
SAT Meter Margin (°F)	58				
#RCP in Service (0-4)	4				
Pressurizer Level (%)	46				
Charging Flow (gpm)	55				
S/G Levels #31 (Wide Range) (%)	55				
#32 (Wide Range) (%)	55				
#33 (Wide Range) (%)	55				
#34 (Wide Range) (%)	55				
S/G Pressure #31 (PSIG)	750				
#32 (PSIG)	750				
#33 (PSIG)	750				
#34 (PSIG)	750				
VC Pressure (PSIG)	0.1				
Aux. FW Flow (Total) (GPM)	0				
Cond.Storage Tank Level (Ft.)	24.2				
VC Temperature (°F)	120				
VC Sump Level Elev (Ft.in.)	5"				
Recirc.Sump Level (Ft.in.)	0				
RWST Level (Ft.)	36.2				
Spray Add. Tank (%)	86				
VC Hydrogen (%)	—				
VC Dew Point (°F)	86				
FCU Weir Level (in.)	6				
Reactor Vessel Level (%)	100				

NOTES:

NOTE: To be provided with message #1A only.

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#1A

PARAMETER			TIME 0830	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<.1				
R-2	VC 80'	(mR/hr)	.08				
R-4	Charging Pump	(mR/hr)	<.1				
R-5	FSB	(mR/hr)	.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	50K				
R-12	VC Gas	(KCPM)	250				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	600K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#1A

PARAMETER	TIME 0830	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

NOTE: To be provided with Message #1A only.

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 0830

#1A

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1	X			
	#32 4	X			
	#33 3	X			
	#34 2	X			
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A		X		
	#33 6A		X		
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A		X		
	#32		X		
	#33 6A		X		
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A		X		
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A		X		
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A		X		
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A		X		
	#32 6A		X		
	#33 5A		X		
	#34 6A		X		

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A		X		
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31		X		
	#32		X		
Comp.Cool Heat Exch.	#31	X			
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation					
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

DATE: March 13, 1991 TIME: 0845

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 2

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	RCS leak of 75 gpm from pressurizer safety valve 464. Radiation Monitor Panel alarms (R-11). Plant Status Log #2	· Operators should assess the source/magnitude of the leak & capability to isolate. See Field Report #1 if R11/12 sample has been requested. · Magnitude is >50 gpm, therefore, declare an ALERT. · Plant shutdown should be initiated (data reflects hot shutdown being achieved in 1 hour). · Offsite notification should follow as per IP-1038. · OSC/TSC activated as per respective TAB in Vol. II. EOF may also be activated at the discretion of the ED. · Initiate site personnel notification & update every 30 minutes. · The plant status log sheets reflect starting an additional charging pump. An anticipated operator action is to accommodate the loss of RCS.	ALERT should be declared within 15 minutes.

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 0845

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 2

The following alarms annunciate:

PRESSURIZER SAFETY VALVE OUTLET HIGH TEMP

PRESSURIZER LOW PRESSURE

PRESSURIZER LOW LEVEL

RADIATION MONITOR PANEL ALARMS

Plant status as per Plant Status Log #2

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#2

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 1 3 1991

PARAMETER		TIME 0845	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	594				
RCS Incore T/C Highest	(°F)	595				
RCS Average Temperature	(°F)	566				
RCS Pressure	(PSIG)	2185				
SAT Meter Margin	(°F)	55				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	44				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	750				
#32	(PSIG)	750				
#33	(PSIG)	750				
#34	(PSIG)	750				
VC Pressure	(PSIG)	0.1				
Aux. FW Flow (Total)	(GPM)	0				
Cond.Storage Tank Level	(Ft.)	24.2				
VC Temperature	(°F)	120				
VC Sump Level Elev	(Ft.in.)	5"				
Recirc.Sump Level	(Ft.in.)	0				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	–				
VC Dew Point	(°F)	86				
FCU Weir Level	(in.)	6				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#2

PARAMETER			TIME 0845	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<.1				
R-2	VC 80'	(mR/hr)	.08				
R-4	Charging Pump	(mR/hr)	<.1				
R-5	FSB	(mR/hr)	.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	250K				
R-12	VC Gas	(KCPM)	255				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#2

PARAMETER	TIME 0845	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 0900

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 3

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #3	The operators may try reducing RCS pressure in an effort to "seat" the safety valve. If so, their efforts will fail. The plant status log data sheets reflect the operators initiating a plant shutdown (with hot shutdown being achieved in one hour).	ALERT

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 0900

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 3

Plant status as per Plant Status Log #3

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#3

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 1 3 1991

PARAMETER		TIME 0900	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	594				
RCS Incore T/C Highest	(°F)	595				
RCS Average Temperature	(°F)	566				
RCS Pressure	(PSIG)	2210				
SAT Meter Margin	(°F)	57				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	46				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	750				
#32	(PSIG)	750				
#33	(PSIG)	750				
#34	(PSIG)	750				
VC Pressure	(PSIG)	0.1				
Aux. FW Flow (Total)	(GPM)	0				
Cond.Storage Tank Level	(Ft.)	24.2				
VC Temperature	(°F)	120				
VC Sump Level Elev	(Ft.in.)	5"				
Recirc.Sump Level	(Ft.in.)	0				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	87				
FCU Weir Level	(in.)	7				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#3

PARAMETER			TIME 0900	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	.08				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	950K				
R-12	VC Gas	(KCPM)	260				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#3

PARAMETER	TIME 0900	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used

DEC 1990

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 0900

3

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1	X			
	#32 4	X			
	#33 3	X			
	#34 2	X			
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A		X		
	#33 6A		X		
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A		X		
	#32		X		
	#33 6A		X		
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A		X		
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A		X		
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A		X		
	#32 6A		X		
	#33 5A		X		
	#34 6A		X		

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A		X		
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31		X		
	#32		X		
Comp.Cool Heat Exch.	#31	X			
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation					
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 0915

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 4

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #4 CCW Surge Tank High Level Alarms - 31 CCW heat exchanger has experienced a leak.	The operators will assess the magnitude & source of the in-leakage. As the component cooling water system (CCW) rad. monitors (R-17a & b) do not indicate an increase in activity, a service water system leakage may be suspected. A Chem. Tech. may be dispatched to sample the system (see Field Report #9). Operator may be dispatched to isolate the heat exchanger (see Field Report #10). TCS personnel may calculate the ability of the CCW system to provide sufficient heat sink.	ALERT

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 0915

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 4

Plant status as per Plant Status Log #4.

CCW Surge Tank High Level Alarm annunciates.

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#4

EP-FORM #31a IP-3 PLANT STATUS LOG DATE: MAR 13 1991

PARAMETER		TIME 0915	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	580				
RCS Incore T/C Highest	(°F)	581				
RCS Average Temperature	(°F)	561				
RCS Pressure	(PSIG)	2235				
SAT Meter Margin	(°F)	72				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	23				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	880				
#32	(PSIG)	880				
#33	(PSIG)	880				
#34	(PSIG)	880				
VC Pressure	(PSIG)	0.1				
Aux. FW Flow (Total)	(GPM)	0				
Cond.Storage Tank Level	(Ft.)	24.2				
VC Temperature	(°F)	120				
VC Sump Level Elev	(Ft.in.)	5"				
Recirc.Sump Level	(Ft.in.)	0				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	88				
FCU Weir Level	(in.)	7				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#4

PARAMETER			TIME 0915	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	.08				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	270				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#4

PARAMETER	TIME 0915	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 0930

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 5

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #5	The five-party phone fails to function. (See Field Report #2) The communicators should inform their Managers. The four-party phone will be used for communication. An I&C Repair/Corrective Action Team should be dispatched with AT&T being notified to repair.	ALERT

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 0930

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 5

Plant status as per Plant Status Log #5

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#5

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 1 3 1991

PARAMETER		TIME 0930	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	558				
RCS Incore T/C Highest	(°F)	559				
RCS Average Temperature	(°F)	551				
RCS Pressure	(PSIG)	2235				
SAT Meter Margin	(°F)	94				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	23				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	930				
#32	(PSIG)	930				
#33	(PSIG)	930				
#34	(PSIG)	930				
VC Pressure	(PSIG)	0.1				
Aux. FW Flow (Total)	(GPM)	0				
Cond.Storage Tank Level	(Ft.)	24.2				
VC Temperature	(°F)	121				
VC Sump Level Elev	(Ft.in.)	5"				
Recirc.Sump Level	(Ft.in.)	0				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	-				
VC Dew Point	(°F)	89				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#5

PARAMETER			TIME 0930	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	.2				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	280				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#5

PARAMETER	TIME 0930	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#5

TIME OF INFORMATION: 0930

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1	X			
	#32 4	X			
	#33 3	X			
	#34 2	X			
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A		X		
	#33 6A		X		
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A		X		
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A		X		
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A		X		
	#32 6A		X		
	#33 5A		X		
	#34 6A		X		

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A		X		
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31		X		
	#32		X		
Comp.Cool Heat Exch.	#31		X		
	#32		X		
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation					
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 0945

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 6

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #6	Hot shutdown is achieved. A cool-down rate of 40° F/hr. is initiated.	ALERT

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 0945

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 6

Plant status as per Plant Status Log #6

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#6

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 1 3 1991

PARAMETER		TIME 0945	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	547				
RCS Incore T/C Highest	(°F)	548				
RCS Average Temperature	(°F)	547				
RCS Pressure	(PSIG)	2200				
SAT Meter Margin	(°F)	100				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	23				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	1000				
#32	(PSIG)	1000				
#33	(PSIG)	1000				
#34	(PSIG)	1000				
VC Pressure	(PSIG)	0.2				
Aux. FW Flow (Total)	(GPM)	140				
Cond.Storage Tank Level	(Ft.)	24.2				
VC Temperature	(°F)	121				
VC Sump Level Elev	(Ft.in.)	11"				
Recirc.Sump Level	(Ft.in.)	11"				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	90				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#6

PARAMETER			TIME 0945	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	.3				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	285				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#6

PARAMETER	TIME 0945	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1000

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 7

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #7	· Plant cooldown continues · TSC & OSC are operational · EOF may be operational if activated by the ED · Notifications being provided from CCR	ALERT

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1000

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 7

Plant status as per Plant Status Log #7

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#7

EP-FORM #31a IP-3 PLANT STATUS LOG DATE: MAR 1 3 1991

PARAMETER		TIME 1000	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	537				
RCS Incore T/C Highest	(°F)	538				
RCS Average Temperature	(°F)	537				
RCS Pressure	(PSIG)	2150				
SAT Meter Margin	(°F)	106				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	23				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	920				
#32	(PSIG)	920				
#33	(PSIG)	920				
#34	(PSIG)	920				
VC Pressure	(PSIG)	0.2				
Aux. FW Flow (Total)	(GPM)	140				
Cond.Storage Tank Level	(Ft.)	24.1				
VC Temperature	(°F)	121				
VC Sump Level Elev	(Ft.in.)	11"				
Recirc.Sump Level	(Ft.in.)	11"				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#7

PARAMETER			TIME 1000	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	.5				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	290				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#7

PARAMETER	TIME 1000	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#7

TIME OF INFORMATION: 1000

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1	X			
	#32 4	X			
	#33 3	X			
	#34 2	X			
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A		X		
	#33 6A		X		
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A		X		
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A		X		
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A		X		
	#32 6A		X		
	#33 5A		X		
	#34 6A		X		

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A		X		
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31		X		
	#32		X		
Comp.Cool Heat Exch.	#31		X		
	#32		X		
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation					
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1015

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 8

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	· Plant Status Log #8 indicates RCS leakage increasing	The data reflect a leak rate increase to 400 gallons per minute and the Engineered Safeguards System actuates. The operators should assess the magnitude of the leak and declare a SITE AREA EMERGENCY due to a loss of coolant exceeding the capacity of two charging pumps. Notification of the upgrade should follow. Accountability should be initiated & completed within thirty minutes. The Emergency Operations Facility will be activitated if it has not been already.	ALERT - SITE AREA to be declared within 15 minutes.

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1015

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 8

Plant status as per Plant Status Log #8

"Pressurizer Low Pressure" alarms

"Pressurizer Low Level" alarms

E.S.S. System activates

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

8

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 1 3 1991

PARAMETER		TIME 1015	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	527				
RCS Incore T/C Highest	(°F)	528				
RCS Average Temperature	(°F)	527				
RCS Pressure	(PSIG)	1235				
SAT Meter Margin	(°F)	41				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	6				
Charging Flow	(gpm)	0				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	840				
#32	(PSIG)	840				
#33	(PSIG)	840				
#34	(PSIG)	840				
VC Pressure	(PSIG)	0.2				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	24				
VC Temperature	(°F)	121				
VC Sump Level Elev	(Ft.in.)	11"				
Recirc.Sump Level	(Ft.in.)	11"				
RWST Level	(Ft.)	36.2				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#8

PARAMETER			TIME 1015	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	.6				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	10				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	300				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#8

PARAMETER	TIME 1015	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1020

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 8A

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	35 Containment Fan Cooler Unit fails to start upon the Engineered Safe-guards System actuation.	The CCR operator players should simulate attempting to start the unit with the CCR switch. This will not start the unit. A team may be dispatched to the breaker to troubleshoot repair. It is anticipated the players will replace the breaker with a spare & start the unit. (See Field Report #3) As the unit is redundant to other operating equipment the players may decide to not pursue corrective action. A team to attain a Containment Building atmospheric sample may be dispatched. (See Field Report #1)	ALERT - SITE AREA to be declared within 10 minutes.

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1020

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 8A

35 Containment Fan Cooler Unit fails to start upon the Engineered Safeguards System actuation.

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1030

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 9

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #9	Data reflects no change in the cooldown while the CCR players implement the Emergency Operating Procedures and verify the automatic actuation of E.S.S. equipment.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1030

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 9

Plant status as per Plant Status Log #9

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

9

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 1 3 1991

PARAMETER		TIME 1030	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	527				
RCS Incore T/C Highest	(°F)	528				
RCS Average Temperature	(°F)	527				
RCS Pressure	(PSIG)	850				
SAT Meter Margin	(°F)	0				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	66				
Charging Flow	(gpm)	0				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	840				
#32	(PSIG)	840				
#33	(PSIG)	840				
#34	(PSIG)	840				
VC Pressure	(PSIG)	0.3				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.9				
VC Temperature	(°F)	122				
VC Sump Level Elev	(Ft.in.)	55"				
Recirc.Sump Level	(Ft.in.)	78"				
RWST Level	(Ft.)	35.5				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#9

PARAMETER			TIME 1030	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	2				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	11				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	345				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	60K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#9

PARAMETER	TIME 1030	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1030

#9

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1	X			
	#32 4	X			
	#33 3	X			
	#34 2	X			
Emergency D/Gs	#31 2A	X			
	#32 6A	X			
	#33 5A	X			
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
RHR Pumps	#31 3A	X			
	#32 6A	X			
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A			X	
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A		X		
	#32 3A		X		
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A		X		
	#35 3A		X		
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31	X			
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	110			
	#32 (GPM)	110			
	#33 (GPM)	110			
	#34 (GPM)	110			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1045

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 10

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #10	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1045

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 10

Plant status as per Plant Status Log #10

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#10

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1045	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	517				
RCS Incore T/C Highest	(°F)	518				
RCS Average Temperature	(°F)	517				
RCS Pressure	(PSIG)	850				
SAT Meter Margin	(°F)	10				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	55				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	760				
#32	(PSIG)	760				
#33	(PSIG)	760				
#34	(PSIG)	760				
VC Pressure	(PSIG)	0.3				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.8				
VC Temperature	(°F)	122				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	34.9				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#10

PARAMETER			TIME 1045	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	2				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	11				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Off scale				
R-12	VC Gas	(KCPM)	395				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#10

PARAMETER	TIME 1045	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1100

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 11

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #11	Plant cooldown and depressurization continues. Accountability will identify one person missing. Initiate a Search & Rescue Team to locate missing person. (See Field Report #4) The EOF should initiate dose projection calculat-ions and discuss protective action recommendations. The plant status log reflects shutdown of the diesel generator and RHR pumps as per EOPs.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1100

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 11

Plant status as per Plant Status Log #11

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#11

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1100	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	507				
RCS Incore T/C Highest	(°F)	507				
RCS Average Temperature	(°F)	507				
RCS Pressure	(PSIG)	850				
SAT Meter Margin	(°F)	20				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	55				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	680				
#32	(PSIG)	680				
#33	(PSIG)	680				
#34	(PSIG)	680				
VC Pressure	(PSIG)	0.3				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.7				
VC Temperature	(°F)	122				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	34.3				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	-				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PAGE 1 of 2

#11

PARAMETER			TIME 1100	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	3				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	12				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	435				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#11

PARAMETER	TIME 1100	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1100

#11

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1	X			
	#32 4	X			
	#33 3	X			
	#34 2	X			
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A			X	
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A		X		
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31	X			
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	90			
	#32 (GPM)	90			
	#33 (GPM)	90			
	#34 (GPM)	90			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991

TIME: 1115

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 12

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #12	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1115

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 12

Plant status as per Plant Status Log #12

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#12

EP-FORM #31a IP-3 PLANT STATUS LOG DATE: MAR 13 1991

PARAMETER		TIME 1115	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	497				
RCS Incore T/C Highest	(°F)	497				
RCS Average Temperature	(°F)	497				
RCS Pressure	(PSIG)	825				
SAT Meter Margin	(°F)	30				
#RCP in Service	(0-4)	4				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	55				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	600				
#32	(PSIG)	600				
#33	(PSIG)	600				
#34	(PSIG)	600				
VC Pressure	(PSIG)	0.3				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.6				
VC Temperature	(°F)	122				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	33.7				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	–				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#12

PARAMETER			TIME 1115	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	4				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	13				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	475				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#12

PARAMETER	TIME 1115	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual Meteorological data is to be used

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1130

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 13

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #13	Plant cooldown and depressurization continues. The plant status log reflects shutting down three RCPs & one SIS pump as per EOPs.	SITE AREA

DATE: March 13, 1991 TIME: 1130

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 13

Plant status as per Plant Status Log #13

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#13

EP-FORM #31a IP-3 PLANT STATUS LOG DATE: MAR 13 1991

PARAMETER	TIME 1130	TIME	TIME	TIME	TIME
RCS Incore T/C Average (°F)	487				
RCS Incore T/C Highest (°F)	488				
RCS Average Temperature (°F)	487				
RCS Pressure (PSIG)	725				
SAT Meter Margin (°F)	17				
#RCP in Service (0-4)	1				
Pressurizer Level (%)	100				
Charging Flow (gpm)	55				
S/G Levels #31 (Wide Range) (%)	55				
#32 (Wide Range) (%)	55				
#33 (Wide Range) (%)	55				
#34 (Wide Range) (%)	55				
S/G Pressure #31 (PSIG)	520				
#32 (PSIG)	520				
#33 (PSIG)	520				
#34 (PSIG)	520				
VC Pressure (PSIG)	0.3				
Aux. FW Flow (Total) (GPM)	130				
Cond.Storage Tank Level (Ft.)	23.5				
VC Temperature (°F)	122				
VC Sump Level Elev (Ft.in.)	OFFSCALE				
Recirc.Sump Level (Ft.in.)	OFFSCALE				
RWST Level (Ft.)	33.1				
Spray Add. Tank (%)	86				
VC Hydrogen (%)	—				
VC Dew Point (°F)	OFFSCALE				
FCU Weir Level (in.)	OFFSCALE				
Reactor Vessel Level (%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#13

PARAMETER			TIME 1130	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	4				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	13				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Off scale				
R-12	VC Gas	(KCPM)	515				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)	6.08E-7				
R-27	Vent Flow Rate	(CFM)	60K				
R-27	Vent Disch.Rate	(uCi/sec)	14				
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#13

PARAMETER	TIME 1130	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1130

#13

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1		X		
	#32 4		X		
	#33 3	X			
	#34 2		X		
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A		X		
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A			X	
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A		X		
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31	X			
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	80			
	#32 (GPM)	80			
	#33 (GPM)	80			
	#34 (GPM)	80			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1140

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 13A

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Radiation Monitoring Panel alarms. Plant vent radiation monitor (R-27) fails.	A repair & corrective action team will be initiated to repair R-27. (See Field Report #5) Dose assessment personnel will use back-up instrumentation for calculation. A team to attain a plant vent sample may be dispatched. (See Field Report #1)	SITE AREA

DATE: March 13, 1991 TIME: 1140

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 13A

Radiation monitoring panel alarms

R-27 "Loss of Communication" failure

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1145

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 14

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #14	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE.

DATE: March 13, 1991 TIME: 1145

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 14

Plant status as per Plant Status Log #14

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#14

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1145	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	477				
RCS Incore T/C Highest	(°F)	478				
RCS Average Temperature	(°F)	477				
RCS Pressure	(PSIG)	700				
SAT Meter Margin	(°F)	23				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	55				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	400				
#32	(PSIG)	400				
#33	(PSIG)	400				
#34	(PSIG)	400				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.4				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	32.5				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	–				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#14

PARAMETER			TIME 1145	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	5				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	14				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	545				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#14

PARAMETER	TIME 1145	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1200

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 15

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #15	Plant cooldown and depressurization continues. Data reflect removing SI pump from service and starting an additional charging pump.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1200

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 15

Plant status as per Plant Status Log #15

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#15

MAR 13 1991

EP-FORM #31a IP-3 PLANT STATUS LOG DATE: _____

PARAMETER		TIME 1200	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	462				
RCS Incore T/C Highest	(°F)	463				
RCS Average Temperature	(°F)	462				
RCS Pressure	(PSIG)	575				
SAT Meter Margin	(°F)	24				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	110				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	400				
#32	(PSIG)	400				
#33	(PSIG)	400				
#34	(PSIG)	400				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.3				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	31.9				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#15

PARAMETER			TIME 1200	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	6				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	15				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	580				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#15

PARAMETER	TIME 1200	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1200

#15

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1		X		
	#32 4		X		
	#33 3	X			
	#34 2		X		
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A	X			
	#33 6A		X		
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A			X	
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31	X			
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	70			
	#32 (GPM)	70			
	#33 (GPM)	70			
	#34 (GPM)	70			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1215

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 16

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #16	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1215

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 16

Plant status as Per Plant Status Log #16

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#16

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1215	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	450				
RCS Incore T/C Highest	(°F)	451				
RCS Average Temperature	(°F)	450				
RCS Pressure	(PSIG)	525				
SAT Meter Margin	(°F)	24				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	380				
#32	(PSIG)	380				
#33	(PSIG)	380				
#34	(PSIG)	380				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	31.7				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#16

PARAMETER			TIME 1215	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	7				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	16				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	610				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#16

PARAMETER	TIME 1215	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1230

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 17

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #17	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1230

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 17

Plant status as per Plant Status Log #17

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

17

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1230	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	437				
RCS Incore T/C Highest	(°F)	438				
RCS Average Temperature	(°F)	437				
RCS Pressure	(PSIG)	450				
SAT Meter Margin	(°F)	18				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	360				
#32	(PSIG)	360				
#33	(PSIG)	360				
#34	(PSIG)	360				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	31.5				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	–				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

#17

DATE: MAR 13 1991

PARAMETER			TIME 1230	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	8				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	17				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	630				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#17

PARAMETER	TIME 1230	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#17

TIME OF INFORMATION: 1230

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1		X		
	#32 4		X		
	#33 3	X			
	#34 2		X		
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A			X	
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31				
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	70			
	#32 (GPM)	70			
	#33 (GPM)	70			
	#34 (GPM)	70			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1240

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 17A

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Zone 183 34' elevation OTSC trouble signal lights and alarms	The TSC Diesel Generator will provide back-up power to the OSC/TSC. (See Field Reports #6, 7) A team may dispatched to monitor diesel operation. The TSC & OSC Managers should have personnel verify their stations are operational. Upon replacement of breaker #5A, see Field Report #8 for shutdown of TSC diesel generation.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1240

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 17A

Zone 183 - 34' Elevation OTSC trouble signal lights and alarms.

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1245

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 18

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #18	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET ISFOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1245

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 18

Plant status as per Plant Status Log #18

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

18

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1245	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	424				
RCS Incore T/C Highest	(°F)	425				
RCS Average Temperature	(°F)	424				
RCS Pressure	(PSIG)	450				
SAT Meter Margin	(°F)	30				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	330				
#32	(PSIG)	330				
#33	(PSIG)	330				
#34	(PSIG)	330				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	130				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	31.3				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	–				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#18

PARAMETER			TIME 1245	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	10				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	19				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	655				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#18

PARAMETER	TIME 1245	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1300

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 19

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #19	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1300

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 19

Plant status as per Plant Status Log #19

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

19

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1300	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	410				
RCS Incore T/C Highest	(°F)	411				
RCS Average Temperature	(°F)	410				
RCS Pressure	(PSIG)	450				
SAT Meter Margin	(°F)	45				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	310				
#32	(PSIG)	310				
#33	(PSIG)	310				
#34	(PSIG)	310				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	100				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	31.1				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#19

PARAMETER			TIME 1300	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	11				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	21				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	680				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#19

PARAMETER	TIME 1300	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1300

#19

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1		X		
	#32 4		X		
	#33 3	X			
	#34 2		X		
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
RHR Pumps	#31 3A		X		
	#32 6A		X		
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A			X	
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31				
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	70			
	#32 (GPM)	70			
	#33 (GPM)	70			
	#34 (GPM)	70			
Low Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1315

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 20

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #20	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1315

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 20

Plant status as per Plant Status Log #20

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#20

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1315	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	395				
RCS Incore T/C Highest	(°F)	396				
RCS Average Temperature	(°F)	395				
RCS Pressure	(PSIG)	450				
SAT Meter Margin	(°F)	60				
#RCP in Service	(0-4)	1				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	300				
#32	(PSIG)	300				
#33	(PSIG)	300				
#34	(PSIG)	300				
VC Pressure	(PSIG)	0.4				
Aux. FW Flow (Total)	(GPM)	100				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	29.9				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#20

PARAMETER			TIME 1315	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	12				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	22				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	700				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#20

PARAMETER	TIME 1315	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

IP-3 PARTIAL DRILL

DATE: March 13, 1990 TIME: 1330

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 21

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #21	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1330

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 21

Plant status as per Plant Status Log #21

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#21

EP-FORM #31a IP-3 PLANT STATUS LOG DATE: MAR 13 1991

PARAMETER		TIME 1330	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	380				
RCS Incore T/C Highest	(°F)	381				
RCS Average Temperature	(°F)	380				
RCS Pressure	(PSIG)	450				
SAT Meter Margin	(°F)	75				
#RCP in Service	(0-4)	0				
Pressurizer Level	(%)	130				
Charging Flow	(gpm)	50				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	280				
#32	(PSIG)	280				
#33	(PSIG)	280				
#34	(PSIG)	280				
VC Pressure	(PSIG)	.4				
Aux. FW Flow (Total)	(GPM)	100				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	29.7				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#21

PARAMETER			TIME 1330	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	13				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	23				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	715				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#21

PARAMETER	TIME 1330	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data not to be used.

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1330

#21

PARAMETER	BUS #	STATUS O	STATUS S	STATUS OS	REMARKS
Reactor Coolant Pumps	#31 1		X		
	#32 4		X		
	#33 3		X		
	#34 2		X		
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A		X		
	#33 6A		X		
RHR Pumps	#31 3A	X			
	#32 6A	X			
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A	X			
	#32		X		
	#33 6A	X			
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A		X		
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	STATUS S	STATUS OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31				
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		Completed			
(Phase A or B Valves which		N/A			
are not in proper position)					
High Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Low Head SIS Flow	#31 (GPM)	1500			
	#32 (GPM)	1500			
	#33 (GPM)	1500			
	#34 (GPM)	1500			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

1140 Initiating Cue

Data will reflect that the Plant Vent Radiation Monitor (R-27) has failed to operate.

Expected Response:

A Repair and Corrective Action Team will be dispatched to repair the monitor. The unit being out of service has no impact on the scenario.

Dose assessment personnel will use back-up radiation monitors.

1240 Initiating Cue

A message will be given to TSC and OSC personnel that all power has been lost in the TSC/OSC except that powered by UPS. Power will be returned after approximately 10 sec. at which time the diesel will pick up the load.

Expected Response:

TSC/OSC personnel should assess that the normal power supply to the Centers has failed and the back-up supply (TSC Diesel Generator) has automatically started to provide power. An Operations Team should be dispatched to the TSC diesel to ensure operation. A Repair and Correction Action Team should be sent to determine the loss of power and provide corrective action.

1345 Initiating Cue

Data will reflect the reactor coolant system is <350°F and RHR can be initiated.

Expected Response:

RHR should be placed in service and cooldown continued to "cold shutdown".

1415 Drill Terminated

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1345

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 22

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #22	Plant cooldown and depressurization continues. RHR is initiated.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1345

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 22

Plant status as per Plant Status Log #22

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

#22

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1345	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	350				
RCS Incore T/C Highest	(°F)	350				
RCS Average Temperature	(°F)	350				
RCS Pressure	(PSIG)	450				
SAT Meter Margin	(°F)	105				
#RCP in Service	(0-4)	0				
Pressurizer Level	(%)	100				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	250				
#32	(PSIG)	250				
#33	(PSIG)	250				
#34	(PSIG)	250				
VC Pressure	(PSIG)	.4				
Aux. FW Flow (Total)	(GPM)	100				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	29.7				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

PAGE 1 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#22

PARAMETER			TIME 1345	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	15				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	25				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	Offscale				
R-12	VC Gas	(KCPM)	735				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

DEC 1990

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#22

PARAMETER	TIME 1345	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1400

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 23

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
CCR	Plant Status Log #23	Plant cooldown and depressurization continues.	SITE AREA

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1400

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 23

Plant status as per Plant Status Log #23

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

23

EP-FORM #31a

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

PARAMETER		TIME 1400	TIME	TIME	TIME	TIME
RCS Incore T/C Average	(°F)	350				
RCS Incore T/C Highest	(°F)	350				
RCS Average Temperature	(°F)	350				
RCS Pressure	(PSIG)	425				
SAT Meter Margin	(°F)	105				
#RCP in Service	(0-4)	0				
Pressurizer Level	(%)	97				
Charging Flow	(gpm)	130				
S/G Levels #31 (Wide Range)	(%)	55				
#32 (Wide Range)	(%)	55				
#33 (Wide Range)	(%)	55				
#34 (Wide Range)	(%)	55				
S/G Pressure #31	(PSIG)	250				
#32	(PSIG)	250				
#33	(PSIG)	250				
#34	(PSIG)	250				
VC Pressure	(PSIG)	.4				
Aux. FW Flow (Total)	(GPM)	0				
Cond.Storage Tank Level	(Ft.)	23.2				
VC Temperature	(°F)	123				
VC Sump Level Elev	(Ft.in.)	OFFSCALE				
Recirc.Sump Level	(Ft.in.)	OFFSCALE				
RWST Level	(Ft.)	29.7				
Spray Add. Tank	(%)	86				
VC Hydrogen	(%)	—				
VC Dew Point	(°F)	OFFSCALE				
FCU Weir Level	(in.)	OFFSCALE				
Reactor Vessel Level	(%)	100				

NOTES:

DEC 1990

EP-FORM #31b

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#23

PARAMETER			TIME 1400	TIME	TIME	TIME	TIME
R-1	CCR	(mR/hr)	<0.1				
R-2	VC 80'	(mR/hr)	13				
R-4	Charging Pump	(mR/hr)	0.1				
R-5	FSB	(mR/hr)	0.4				
R-6	Sample Room	(mR/hr)	1				
R-7	VC Seal Table	(mR/hr)	23				
R-8	Drum Station	(mR/hr)	1				
R-10	Steam Line Penetr.	(R/hr)	<0.01				
R-11	VC Part	(KCPM)	offscale				
R-12	VC Gas	(KCPM)	730				
R-13	Vent Part	(KCPM)	400				
R-14	Vent Gas	(KCPM)	2K				
R-15	Air Ejector	(CPM)	25				
R-16	FC Water	(KCPM)	28K				
R-17a	Component Cooling	(CPM)	300				
R-17b	Component Cooling	(CPM)	400				
R-18	Liquid Waste (eff)	(KCPM)	6K				
R-19	S/G Blowdown	(uCi/cc)	8.7E-7				
R-23	FC Water	(KCPM)	15K				
R-25	VC High Range 1-E8	(R/hr)	<1				
R-26	VC High Range	(R/hr)	<1				
R-27	Vent Monitor	(uCi/cc)					
R-27	Vent Flow Rate	(CFM)					
R-27	Vent Disch.Rate	(uCi/sec)					
R-59	RAMS Gas Eff.	(uCi/sec)	3				
R-62A/32	Main Steam	(uCi/cc)	2.05E-4				
R-62B/31	Main Steam	(uCi/cc)	4.10E-4				
R-62C/33	Main Steam	(uCi/cc)	2.05E-4				
R-62D/34	Main Steam	(uCi/cc)	8.2E-4				

EP-FORM #31b

PAGE 2 of 2

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

#23

PARAMETER	TIME 1400	TIME	TIME	TIME	TIME
R-63A/B Failed Fuel (uCi/cc)	1.1E-1				
R-64 55' PAB (mR/hr)	0.1				
R-65 73' PAB (mR/hr)	0.1				
R-66 34' PAB (mR/hr)	0.1				
R-67 41' PAB (mR/hr)	0.2				
R-68 15' PAB (mR/hr)	0.2				
R-69 54' Pipe Pen (mR/hr)	6				
R-70 80' Fan House (mR/hr)	0.1				
Wind Speed (m/sec)					
Wind Direction (degrees)					
Pasquill Category (A-G)					
General Weather Conditions					

NOTES: Actual meteorological data is to be used.

DEC 1990

EP-FORM #31c

IP-3 PLANT STATUS LOG

DATE: MAR 13 1991

TIME OF INFORMATION: 1400

#23

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Reactor Coolant Pumps	#31 1		X		
	#32 4		X		
	#33 3		X		
	#34 2		X		
Emergency D/Gs	#31 2A		X		
	#32 6A		X		
	#33 5A		X		
Offsite Power Avail.	138KV	X			
	13.8KV		X		
SIS Pumps	#31 5A		X		
	#32 2A		X		
	#33 6A		X		
RHR Pumps	#31 3A	X			
	#32 6A	X			
Recirc. Pumps	#31 5A		X		
	#32 6A		X		
Aux.Boiler Feed Pumps	#31 3A		X		
	#32		X		
	#33 6A		X		
Fan Cooler Units	#31 5A	X			
	#32 2A	X			
	#33 5A	X			
	#34 3A	X			
	#35 6A		X		
Cont. Spray Pumps	#31 5A		X		
	#32 6A		X		
Charging Pumps	#31 5A	X			
	#32 3A	X			
	#33 6A		X		
Component Cool Pumps	#31 5A	X			
	#32 2A	X			
	#33 6A	X			
Aux.Comp. Cool Pumps	#31 5A	X			
	#32 6A	X			
	#33 5A	X			
	#34 6A	X			

PARAMETER	BUS #	STATUS O	S	OS	REMARKS
Service Water Pumps	#31 5A	X			
(Essential Header)	#32 2A	X			
	#33 6A	X			
	#34 5A	X			
	#35 3A	X			
	#36 6A		X		
RHR Heat Exchange	#31	X			
	#32	X			
Comp.Cool Heat Exch.	#31				
	#32	X			
Hydrogen Recombiner	#31 5A		X		
	#32 6A		X		
VC Isolation		completed N/A			
(Phase A or B Valves which are not in proper position)					
High Head SIS Flow	#31 (GPM)	0			
	#32 (GPM)	0			
	#33 (GPM)	0			
	#34 (GPM)	0			
Low Head SIS Flow	#31 (GPM)	1500			
	#32 (GPM)	1500			
	#33 (GPM)	1500			
	#34 (GPM)	1500			
Accum. Level	#31 (%)	20			
	#32 (%)	20			
	#33 (%)	20			
	#34 (%)	20			
Gas Turbines	GT-1		X		
(Call Con Edison)	GT-2		X		
	GT-3		X		
Appendix 'R' D/G			X		

THIS FORM TO BE FILLED OUT AND SENT BY THE CONTROL ROOM. S = Standby O = Operating OS = Out of Service

DEC 1990

IP-3 PARTIAL DRILL

DATE: March 13, 1991 TIME: 1415

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 24

ISSUED TO:	SUMMARY OF MESSAGE:	ANTICIPATED RESULTS COMMENTS	EMERGENCY CLASSIFICATION:
Emergency Director	Drill Terminated	Communicate to the Site that the drill is terminated. Facility self critiques. Observers & Controllers prepare notes.	N/A

NOTE: INFORMATION ON THIS SHEET IS FOR CONTROLLER USE ONLY.

DATE: March 13, 1991 TIME: 1415

INDIAN POINT NO. 3 SCENARIO

MESSAGE NUMBER: 24

The drill is terminated.

NOTE: THIS PAGE WILL BE HANDED TO EXERCISE PLAYERS.

- THIS IS A DRILL -

SECTION 6

FIELD REPORTS

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #1

TIME: TIME CONTAINMENT, RCS OR PLANT VENT ACTIVITY IS SAMPLED

LOCATION: HP/CHEMISTRY COUNTING ROOM

INFORMATION TO BE PROVIDED TO: HP/CHEMISTRY TECHNICIANS

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE: N/A

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Attached are the data for Containment (R-11/12) Activity, RCS Activity and Plant Vent Concentration. See the dedicated time for the correct data.

CONTAINMENT ACTIVITY

MARCH 13, 1991 0930

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	1.0e-06
Xe-131M	6.4e-06
Xe-133	5.1e-04
Xe-133M	5.4e-06
Xe-135	7.7e-06
Xe-135M	1.4e-07
Xe-138	1.4e-07
Kr-85M	6.2e-07
Kr-88	9.5e-07
I-132	1.2e-07
I-133	1.0e-07
I-134	1.0e-07
I-135	1.3e-07
TOTAL	5.3e-04

CONTAINMENT ACTIVITY

MARCH 13, 1991 1030

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	1.6e-06
Xe-131M	6.4e-06
Xe-133	5.3e-04
Xe-133M	5.4e-06
Xe-135	1.7e-05
Xe-135M	4.5e-07
Kr-85M	1.7e-06
Kr-88	4.3e-06
I-131	1.2e-07
I-132	3.9e-07
I-133	4.4e-07
I-134	2.0e-07
I-135	5.1e-07
TOTAL	5.6e-04

CONTAINMENT ACTIVITY

MARCH 13, 1991 1130

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	3.6e-06
Xe-131M	6.4e-06
Xe-133	5.9e-04
Xe-133M	5.4e-06
Xe-135	3.6e-05
Xe-135M	7.6e-07
Kr-85M	3.7e-06
Kr-88	8.5e-06
I-131	3.5e-07
I-132	8.4e-07
I-133	1.2e-06
I-134	2.7e-07
I-135	1.3e-06
TOTAL	6.5e-04

CONTAINMENT ACTIVITY

MARCH 13, 1991 1230

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	4.0e-06
Xe-131M	6.4e-06
Xe-133	6.4e-04
Xe-133M	5.4e-06
Xe-135	4.9e-05
Xe-135M	7.9e-07
Kr-85M	4.6e-06
Kr-88	9.9e-06
I-131	5.3e-07
I-132	9.3e-07
I-133	1.8e-06
I-134	1.8e-07
I-135	1.8e-06
TOTAL	7.1e-04

CONTAINMENT ACTIVITY

MARCH 13, 1991 1330

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	3.0e-06
Xe-131M	6.4e-06
Xe-133	6.7e-04
Xe-133M	5.4e-06
Xe-135	6.2e-05
Xe-135M	7.8e-07
Kr-85M	5.0e-06
Kr-88	9.7e-06
I-131	6.5e-07
I-132	8.7e-07
I-133	2.2e-06
I-134	1.0e-07
I-135	2.0e-06
TOTAL	7.5e-04

RCS ACTIVITY

MARCH 13, 1991 0930

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	4.5e-03
Xe-133	6.0e-02
Xe-135	2.5e-02
Xe-135M	4.3e-03
Xe-138	1.1e-02
Kr-85M	3.4e-03
Kr-88	9.6e-03
FISSION GAS TOTAL	1.2e-01
Na-24	2.0e-03
Mn-54	2.1e-03
Co-58	8.4e-04
ACTIVATION PRODUCT TOTAL	5.0e-03
Rb-88	2.8e-02
Rb-89	5.7e-03
I-131	2.3e-03
I-132	1.2e-02
I-133	8.6e-03
I-134	1.5e-02
I-135	1.2e-02
Y-88	3.4e-03
Cs-134	4.4e-03
Cs-137	2.7e-03
Cs-138	1.8e-02
FISSION PRODUCT TOTAL	1.1e-01
TOTAL ACTIVITY	2.4e-01

RCS ACTIVITY

MARCH 13, 1991 1030

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	3.2e-03
Xe-133	5.6e-02
Xe-135	2.2e-02
Xe-135M	5.1e-04
Xe-138	1.1e-03
Kr-85M	2.8e-03
Kr-88	7.5e-03
FISSION GAS TOTAL	9.4e-02
Na-24	1.8e-03
Mn-54	2.0e-03
Co-58	7.9e-04
ACTIVATION PRODUCT TOTAL	4.6e-03
Rb-88	4.6e-03
Rb-89	6.6e-04
I-131	2.2e-03
I-132	9.3e-03
I-133	7.9e-03
I-134	8.0e-03
I-135	1.0e-02
Y-88	3.1e-03
Cs-134	4.1e-03
Cs-137	2.5e-03
Cs-138	6.5e-03
FISSION PRODUCT TOTAL	5.9e-02
TOTAL ACTIVITY	1.6e-01

RCS ACTIVITY

MARCH 13, 1991 1130

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	1.8e-03
Xe-133	4.7e-02
Xe-135	1.7e-02
Xe-135M	2.6e-05
Xe-138	5.3e-05
Kr-85M	2.0e-03
Kr-88	4.9e-03
FISSION GAS TOTAL	7.3e-02
Na-24	1.5e-03
Mn-54	1.6e-03
Co-58	6.6e-04
ACTIVATION PRODUCT TOTAL	3.8e-03
Rb-88	3.8e-04
Rb-89	3.5e-05
I-131	1.8e-03
I-132	5.8e-03
I-133	6.4e-03
I-134	3.0e-03
I-135	7.5e-03
Y-88	2.6e-03
Cs-134	3.4e-03
Cs-137	2.1e-03
Cs-138	1.5e-03
FISSION PRODUCT TOTAL	3.5e-02
TOTAL ACTIVITY	1.1e-01

RCS ACTIVITY

MARCH 13, 1991 1230

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	1.1e-03
Xe-133	4.0e-02
Xe-135	1.4e-02
Xe-135M	1.4e-06
Xe-138	2.5e-06
Kr-85M	1.5e-03
Kr-88	3.3e-03
FISSION GAS TOTAL	6.0e-02
Na-24	1.2e-03
Mn-54	1.4e-03
Co-58	5.7e-04
ACTIVATION PRODUCT TOTAL	3.2e-03
Rb-88	3.3e-05
Rb-89	1.9e-06
I-131	1.5e-03
I-132	3.7e-03
I-133	5.3e-03
I-134	1.2e-03
I-135	5.8e-03
Y-88	2.3e-03
Cs-134	3.0e-03
Cs-137	1.8e-03
Cs-138	3.6e-04
FISSION PRODUCT TOTAL	2.5e-02
TOTAL ACTIVITY	8.8e-02

RCS ACTIVITY

MARCH 13, 1991 1330

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	6.8e-04
Xe-133	3.7e-02
Xe-135	1.2e-02
Xe-135M	8.3e-08
Xe-138	1.3e-07
Kr-85M	1.2e-03
Kr-88	2.4e-03
FISSION GAS TOTAL	5.3e-02
Na-24	1.1e-03
Mn-54	1.3e-03
Co-58	5.3e-04
ACTIVATION PRODUCT TOTAL	2.9e-03
Rb-88	3.0e-06
Rb-89	1.1e-07
I-131	1.4e-03
I-132	2.5e-03
I-133	4.8e-03
I-134	5.0e-04
I-135	4.9e-03
Y-88	2.1e-03
Cs-134	2.8e-03
Cs-137	1.7e-03
Cs-138	9.3e-05
FISSION PRODUCT TOTAL	2.1e-02
TOTAL ACTIVITY	7.7e-02

PLANT VENT CONCENTRATION

MARCH 13, 1991 0000

Results are less than MDA.

NOTE to observer/controller:

Results of any plant vent samples taken during the drill are less than MDA.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #2

TIME: 0930 & WHEN THE EOF IS STAFFED

LOCATION: CR/OSC/TSC/EOF

INFORMATION TO BE PROVIDED TO: 5 PARTY PHONE COMMUNICATORS

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE:

The five party line phone is inoperable. There is no visible problem with the phone or phone line.

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Ensure message is transmitted to communicators prior to them picking up the phone. The four party line should be used as the back-up. AT&T should be notified by plant personnel (simulate). Upon call, controller will advise that repair will take approximately 5-6 hours.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #3

TIME: 1020

LOCATION: 15' CONTROL BUILDING

INFORMATION TO BE PROVIDED TO:

Repair & Corrective Action Team investigating 35 Containment Fan Cooler Unit failure

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE:

Bus 6A, breaker 12D appears normal, but there is indication that the latching spring is not charged.

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Spring will not charge, therefore breaker will not close and requires replacement. Once breaker is replaced, power can be restored.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #4

TIME: 1100 UNTIL FOUND

LOCATION: CONDENSATE POLISHER BUILDING

INFORMATION TO BE PROVIDED TO: SEARCH & RESCUE TEAM

GENERAL AREA RAD. LEVELS: AS READ

SPECIFIC AREA RAD. LEVELS: AS READ

VISUAL DESCRIPTION AT SCENE:

Chemistry Tech. is working in the Condensate Polisher Building

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Ensure that the OSC is notified that the missing person is found.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #5

TIME: 1140

LOCATION: 33' CONTROL BUILDING

INFORMATION TO BE PROVIDED TO:

REPAIR & CORRECTIVE ACTION TEAM INVESTIGATING R-27 FAILURE

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE:

The AC power supply fuse (F6) to RM80 is blown.

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Replacing the 2.5 amp fuse will correct the problem.

Once R-27 has been verified operational, ensure Control Room Controller is notified.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #6

TIME: 1240

LOCATION: TSC/OSC

INFORMATION TO BE PROVIDED TO: TSC/OSC STAFF

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE:

All power supplied to the TSC/OSC is lost for approximately 10 secs. After the 10 secs., CP-42 is alarming, UPS panel by the computer room door is alarming, diesel control panel is alarming with the emergency power light lit, and Fire Control Panel Zones 1-7 are alarming.

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Ensure that staff is cognizant of alarms and the fact that they were caused by the TSC's transfer to the diesel power supply. All alarms should be acknowledged and cleared with the exception of the Diesel Control Panel.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #7

TIME: 1240

LOCATION: 15' TURBINE BUILDING

INFORMATION TO BE PROVIDED TO:

REPAIR & CORRECTIVE ACTION TEAM INVESTIGATING BUS 313-BREAKER 5A FAILURE

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE:

Bus 313-Breaker 5A appears normal, but there's indication that the latching spring is not charged.

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Spring charges but the breaker will not latch closed. Breaker replacement will restore power to the breaker.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #8

TIME: AFTER REPLACEMENT OF BREAKER #5A

LOCATION: TSC DIESEL ROOM

INFORMATION TO BE PROVIDED TO:

INDIVIDUAL/TEAM ASSIGNED TO SHUT DOWN TSC DIESEL

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE:

TSC Diesel is running and Bus 313, breaker 5A has been replaced and is ready to supply normal power to MCC K.

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Ensure that the normal power supply to MCC K is re-established and that the TSC Diesel is shutdown in accordance with SOP-TSC-2.

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #9

TIME: 0915

LOCATION: 4TH FLOOR ADMIN. BLDG. RADIO CHEM. LAB

INFORMATION TO BE PROVIDED TO: CHEM. TECH.

GENERAL AREA RAD. LEVELS: N/A

SPECIFIC AREA RAD. LEVELS: N/A

VISUAL DESCRIPTION AT SCENE: N/A

INSTRUCTIONS TO CONTROLLER/OBSERVER:

Listed below is the Component Cooling water sample analysis:

Component Cooling Water

Equilibrium:

pH	9.69
MoO_4	295 ppm
NO_2	1060 ppm
Cr	.284 ppm
Activity	4.3 E-6

After Leak:

pH	9.20
MoO_4	260 ppm
NO_2	900 ppm
Cr	.20 ppm
Activity	3.8 E-6

1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

FIELD REPORT #10

TIME: 0915

LOCATION: WHUT CELL/COMPONENT COOLING WATER SURGE TANKS (CCWST)/ COMPONENT COOLING WATER HEAT EXCHANGER (CCWHx)

INFORMATION TO BE PROVIDED TO: NPO

GENERAL AREA RAD. LEVELS: AS READ

SPECIFIC AREA RAD. LEVELS: AS READ

VISUAL DESCRIPTION AT SCENE: N/A

INSTRUCTIONS TO CONTROLLER/OBSERVER:

The NPO may be instructed to do various tasks:
- isolate the leak via 31 or 32 CCWHx
- drain either CCWST
- manually tie bottom connections with 32 & 33 WHUT'S
- other activities as directed by the Control Room

Note: If 32 CCWHx is isolated, the WHUT water level continues to increase.
If 31 CCWHx is isolated, the WHUT water level remains constant and if the NPO drains the CCWST, the water will return back to normal level.

Also, the Control Room controller must be notified if #31 CCWHx remains isolated or not.

SECTION 7

RADIOLOGICAL DATA

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

SECTION 7

INDIAN POINT NO. 3 NUCLEAR POWER PLANT
1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

I. OVERVIEW

This scenario sequence does not incorporate a release of radiological material affecting areas outside of the Radiologically Controlled Area. As such, real-time ("as read") data will be utilized for the following parameters/ indications:

- In-Plant Surveys
- Site Perimeter Surveys
- Reuter-Stokes Monitors
- Offsite Surveys
- Assembly Area Readings
- Technical Support Center Radiation Monitor Readings

Participants are expected to follow appropriate procedures and demonstrate their ability to obtain such data. Once they have done this, they will be advised that the "as read" data is the information on which to base their response actions.

II. EQUILIBRIUM ISOTOPIC DISTRIBUTIONS

a. RCS EQUILIBRIUM ACTIVITY

MARCH 13, 1991 0000

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	4.7e-03
Xe-133	6.3e-02
Xe-135	2.6e-02
Xe-135M	4.5e-03
Xe-138	1.1e-02
Kr-85M	3.5e-03
Kr-88	1.0e-02
FISSION GAS TOTAL	1.2e-01
Na-24	2.1e-03
Mn-54	2.2e-03
Co-58	8.8e-04
ACTIVATION PRODUCT TOTAL	5.2E-03
Rb-88	2.9e-02
Rb-89	5.9e-03
I-131	2.4e-03
I-132	1.3e-02
I-133	9.0e-03
I-134	1.6e-02
I-135	1.2e-02
Y-88	3.5e-03
Cs-134	4.6e-03
Cs-137	2.8e-03
Cs-138	1.9e-02
FISSION PRODUCT TOTAL	1.2e-01
TOTAL ACTIVITY	2.5e-01

b. CONTAINMENT EQUILIBRIUM ACTIVITY

MARCH 13, 1991 0000

ISOTOPE	CONCENTRATION UCi/CC
Ar-41	6.0e-07
Xe-131M	6.4e-06
Xe-133	5.0e-04
Xe-133M	5.4e-06
Xe-135	4.9e-06
Kr-85M	2.6e-07
TOTAL	5.2e-04

SECTION 8

METEOROLOGICAL INFORMATION

NEW YORK POWER AUTHORITY

INDIAN POINT NO. 3 NUCLEAR POWER PLANT

1991 NRC OBSERVED PARTIAL EXERCISE

MARCH 13, 1991

SECTION 8

INDIAN POINT NO. 3 NUCLEAR POWER PLANT
1991 NRC OBSERVED PARTIAL EXERCISE
MARCH 13, 1991

METEOROLOGICAL INFORMATION

This scenario requires the use of real-time meteorological data and forecast information. As such, participants are expected to follow appropriate procedures and utilize actual equipment in order to obtain such information.

The procedures related to obtaining such data are IP-1003 and IP-1004. The equipment which may be utilized includes the Meteorological Display Panel in the Control Room, telephone data links to the meteorological tower and the MIDAS computer, and the MIDAS computer in the EOF.