Indian Point 3 Nuclear Power Plant P.O. Box 215 Buchanan, New York 10511 914 736.8001



Joseph E. Russell Resident Manager

July 20, 1992 IP3-NRC-92-048

License No. 50-286 Docket No. DPR-64

Mr. Curtis J. Cowgill, Chief Reactor Projects Branch No. 1 U.S. Nuclear Regulatory Commission Region I 475 Allendale Road King of Prussia, PA 19406

Subject: Status of EDSFI (Inspection Report No. 50-286/91-80) Action Items

Dear Mr. Cowgill:

On October 21, 1991, letter IP3-NRC-91-062 provided a schedule for the subject action items. Due to 1992 outage emergent work and expanded scope of previously identified tasks, several of the items have been rescheduled per the attached table. Items not tabulated are complete, or on the original schedule.

Should you or your staff have any questions concerning this matter, please contact Mr. M. Peckham at 914-736-8041.

Very truly yours,

K. anel

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PDR

Joseph E. Russell Resident Manager Indian Point 3 Nuclear Power Plant

jer/we/rj Attachment

cc: U.S. Nuclear Regulatory Commission (original) Attn: Document Control Desk Mail Station P1-137 Washington, DC 20555

> IP3 Resident Inspector Indian Point 3 U.S. Nuclear Regulatory Commission P.O. Box 337 Buchanan, New York 10511

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Extended Schedule for Completion of Actions Identified in NRC Inspection Report 50-286/91-80

### 1. Item 91-80-01: Loading of EDS Buses

Action: The 120 VAC sizing calculations, including the 120 VAC load study, will be issued. Preliminary results indicate that load values are within the equipment ratings. To continue to eliminate overly conservative values from the load study, the Authority will obtain nameplate data from normally energized equipment made accessible during the next refueling outage.

<u>Original Completion Dates:</u> Issue preliminary sizing calculations and load study - October 31, 1991. Revise sizing calculations and load study -July 30, 1992.

<u>Revised Completion Date:</u> The preliminary sizing calculations and load study results were not issued. The final sizing calculations and load study, which incorporates nameplate data obtained during the 1992 refueling outage, will be completed by October 31, 1992.

#### 2.

# Item 91-80-07: 120 VAC Voltage Drop Calculation

Action: The Authority will obtain nameplate data from normally energized 120 VAC equipment when accessible during the cycle 8/9 refueling outage. The 120 VAC voltage drop calculations will be completed subsequent to obtaining the nameplate data.

Original Completion Date: June 30, 1992.

<u>Revised Completion Date:</u> Voltage drop calculations will follow the 120 VAC load study results (item 91-80-01) - December 31, 1992.

# 3. Item 91-80-08: 480 VAC Switchgear Fault Analysis

Action: Determine whether it is necessary to modify the breakers for higher short circuit current ratings. (Note: The first action of the original item 91-80-08, to prevent a three-phase bolted fault from a single failure, has been completed).

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## Original Completion Date: End of 1991.

<u>Revised Completion Date:</u> System design modifications which may impact this decision are scheduled for completion December 15, 1992. At that time a breaker capacity study will be scheduled.

## I. Item 91-80-12: AC Systems Coordination

Action: Preliminary computerized 480 VAC system short circuit calculations have been performed. Once finalized, these calculations will be used to perform an AC system coordination study.

Original Completion Date: July 6, 1992.

<u>Revised Completion Date:</u> This work is ongoing with scheduled completion September 30, 1992.

Action: The 120 VAC system coordination study will be reevaluated (and revised as necessary) based on nameplate data obtained during the 1992 refueling outage.

Original Completion Date: June 30, 1992.

Revised Completion Date: October 31, 1992.