

December 11, 2009

NRC 2009-0129
10 CFR 50.90

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Units 1 and 2
Dockets 50-266 and 50-301
Renewed License Nos. DPR-24 and DPR-27

License Amendment Request 261
Extended Power Uprate
Technical Specification 5.6.4, Core Operating Limits Report (COLR) Mark-up

- References:
- (1) FPL Energy Point Beach, LLC letter to NRC, dated April 7, 2009, License Amendment Request 261, Extended Power Uprate (ML091250564)
 - (2) FPL Energy Point Beach, LLC letter to NRC, dated November 25, 2008, License Amendment Request 258, Incorporate Best Estimate Large Break Loss of Coolant Accident (LOCA) Analyses Using ASTRUM (ML083330160)
 - (3) NRC letter to NextEra Energy Point Beach, LLC, dated October 29, 2009, Point Beach Nuclear Plant, Units 1 and 2 – Issuance of Amendments RE: ASTRUM Implementation for Large-Break LOCA Analysis (TAC Nos. ME0170 and ME0171) (ML092931684)
 - (4) NUREG-1431, Revision 3, dated June 30, 2004, Standard Technical Specifications Westinghouse Plants (ML041830612 and ML041830205)

NextEra Energy Point Beach, LLC (NextEra) submitted License Amendment Request (LAR) 261 (Reference 1) to the NRC pursuant to 10 CFR 50.90. The proposed amendment would increase each unit's licensed thermal power level from 1540 megawatts thermal (MWt) to 1800 MWt, and revise the Technical Specifications to support operation at the increased thermal power level.

NextEra requested to amend its Renewed Licenses (Reference 2) in support of LAR 261 by incorporating new large break loss of coolant accident (LBLOCA) analyses using the Automated Statistical Treatment of Uncertainty Method (ASTRUM). Via Reference (3), the NRC approved the ASTRUM license amendment request.

Enclosure 1 of this submittal contains proposed mark-ups to Technical Specification (TS) 5.6.4, Core Operating Limits Report (COLR). The enclosed TS revision incorporates the NRC approval of the ASTRUM methodology (Reference 3) at Point Beach Nuclear Plant, and also corrects administrative errors contained in References 15, 16 and 17 of TS 5.6.4, provided in Reference (1). The mark-up now reflects NUREG-1431 (Reference 4) reference citations. The enclosed mark-up to TS 5.6.4 replaces the proposed mark-up provided in Reference (1).

Summary of Regulatory Commitments

The Technical Specification mark-ups provided in this letter fulfill the following Regulatory Commitment in Reference (1):

- Final TS 5.6.4, Core Operating Limits Report, markups will be submitted within 45 days of Commission approval of LAR 258, Incorporate Best Estimate Large Break Loss-of-Coolant Analyses (LBLOCA).


The information contained in this letter does not alter the no significant hazards consideration contained in Reference (1) and continues to satisfy the criteria of 10 CFR 51.22 for categorical exclusion from the requirements for an environmental assessment.

In accordance with 10 CFR 50.91, a copy of this letter is being provided to the designated Wisconsin Official.

I declare under penalty of perjury that the foregoing is true and correct.
Executed on December 11, 2009.

Very truly yours,

NextEra Energy Point Beach, LLC



Larry Meyer - Active Site V.P.
FOR

Larry Meyer
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
PSCW

ENCLOSURE 1

**NEXTERA ENERGY POINT BEACH, LLC
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2**

**LICENSE AMENDMENT REQUEST 261
EXTENDED POWER UPRATE**

**TECHNICAL SPECIFICATION 5.6.4, CORE OPERATING
LIMITS REPORT (COLR) MARK-UP**

5.6 Reporting Requirements

5.6.4 CORE OPERATING LIMITS REPORT (COLR) (continued)

Insert
Reference 4
from next page

- (4) ~~WCAP-14787-P, Rev. 2, "Revised Thermal Design Procedure Instrument Uncertainty Methodology for Wisconsin Electric Power Company Point Beach Units 1 & 2 (Fuel Upgrade & Update to 1656 MWt NSSS Power with Feedwater Venturis, or 1679 MWt NSSS Power with LEFM on Feedwater Header), October, 2002 (approved by NRC Safety Evaluation, November 29, 2002).~~
- (5) WCAP-10054-P-A, "Westinghouse Small Break ECCS Evaluation Model Using The NOTRUMP Code," August 1985.
- (6) WCAP-10054-P-A, "Addendum to the Westinghouse Small Break ECCS Evaluation Model Using the NOTRUMP Code: Safety Injection into the Broken Loop and COSI Condensation Model," Addendum 2, Revision 1, July 1997.
- (7) WCAP-8745-P-A, "Design Bases for the Thermal Overpower ΔT and Thermal Overtemperature ΔT Trip Functions," September 1986.
- (8) ~~WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control," Revision 1A, February 1994. DELETED~~
- (9) WCAP-10924-P-A, "Large Break LOCA Best Estimate Methodology, Volume 2: Application to Two-Loop PWRs Equipped with Upper Plenum Injection," and Addenda, December 1988. (cores not containing 422 V+ fuel)
- (10) WCAP-10924-P-A, "LBLOCA Best Estimate Methodology: Model Description and Validation: Model Revisions," Volume 1, Addendum 4, August 1990. (cores not containing 422 V+ fuel)
- (11) Caldon, Inc., Engineering Report-80P, "TOPICAL REPORT: Improving Thermal Power Accuracy and Plant Safety While Increasing Operating Power Level Using the LEFM[✓]™ System," Revision 0, March 1997.
- (12) Caldon, Inc., Engineering Report-160P, "Supplement to Topical Report ER-80P: Basis for a Power Update With the LEFM[✓]™ System," Revision 0, May 2000.
- (13) WCAP-16009-P-A, "Realistic Large-Break LOCA Evaluation Methodology Using the Automated Statistical Treatment of Uncertainty Method (ASTRUM)," Revision 0, January 2005.


Insert
References
14 – 17 from
next page

- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

INSERT-1

4. WCAP-14787, Rev. 3, "Westinghouse Revised Thermal Design Procedure Instrument Uncertainty Methodology for Point Beach 1 & 2 Power Uprate (1775 MWt - Core Power with Feedwater Venturis, or 1800 MWt - Core Power with LEFM on Feedwater Header)."

Previously submitted in LAR 241
(ML083450683)



INSERT-2

14. WCAP-16259-P-A, "Westinghouse Methodology for Application of 3-D Transient Neutronics to Non-LOCA Accident Analysis." August 2006.
15. WCAP-8403 (nonproprietary), "Power Distribution Control and Load Following Procedures," Westinghouse Electric Corporation, September 1974.
16. NS-TMA-2198, Westinghouse to NRC Letter, Attachment, "Operation and Safety Analysis Aspects of an Improved Load Follow Package," January 31, 1980.
17. NS-CE-687, Westinghouse to NRC Letter, July 16, 1975.