OPERATING DATA REPORT

Docket No.	50-286
Date	01-04-88
Completed By	L. Kelly
Telephone914-	736-8340

OPERATING STATUS

1.	Unit Name: Indian Point No. 3 Nuclear Power	Plant Notes	• •	
2.	Reporting Period: December 1987			
3.	Licensed Thermal Power (MWt): 3025	· · · ·		•
4	Namenlate Rating (Gross MWa): 1013	•		
5	Decign Floatrical Pating (Not Muc): 96	5	·.	
۶. د	Mentium Dependentie (Crease Mile): 1000	<u></u>	· · · · ·	
0. 7	Maximum Dependable Capacity (Gloss Hwe): 1000	U	· · · · · ·	
/•	Maximum Dependable Capacity (Net Mwe): 90.	<u> </u>		
8.	If Changes Occur in Capacity Ratings (Items) Give Reasons:	Number 3 through	7) Since Last R	eport.
0	Deven Level to Uhich Destricted If Any (Not	+ MUa) ·	· · · · · · · · · · · · · · · · · · ·	·
9. 10.	Reasons for Restrictions, If Any:	L Mwe):	· ····	
			<u></u>	<u> </u>
		<u> </u>		
·		This Month	Yr. to Date	Cumulative
11.	Hours In Reporting Period	744	8.760	99.385
12.	Number of Hours Reactor Was Critical	715.70	5,496,4	59,345.6
13.	Reactor Reserve Shutdown Hours	0	0	0
14.	Hours Generator On-Line	709.83	5,399,9	57.461.5
15	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	2.062.215	15,562,101	154,094,962
17	Gross Electrical Energy Generated (MWH)	681,660	5,033,590	48,338,205
18.	Net Electrical Generated (MWH)	658,111	4,850,586	46,390,866
19.	Unit Service Factor	95.4	61.6	57.8
20.	Unit Availability Factor	95.4	61.6	57.8
21	Unit Capacity Factor (Using MDC Net)	91.7	58.4 *	50.3 *
22.	Unit Capacity Factor (Using DER Net)	91.7	57.4	48.4
23.	Unit Forced Outage Rate	4.6	2.6	18.6
-51			· · · · · · · · · · · · · · · · · · ·	,
24.	Shutdowns Scheduled Over Next 6 Months(Type	,Date, and Duratio	on of Each): * W	eighted Average
		· · · · · · · · · · · · · · · · · · ·	· · · ·	
			· · · · · · · · · · · · · · · · · · ·	
25.	If Shut Down At End Of Report Period. Estima	ated Date of Star	tup:	
26.	Units In Test Status (Prior to Commercial Op INITIAL CRITICALITY	peration): For	ecast Ach	ieved
	INITIAL ELECTRICITY		· · · ·	
	COMMERCIAL OPERATION	·	<u> </u>	
	COTTIENCIAL OF ENALION	· .		
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-286
UNIT	IP-3
DATE	01-04-88
COMPLETED BY	L. Kelly
TELEPHONE (914) 736-8340

MONI	'H	December 1987	-	•	
	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	e de la constant de l	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
•	1	972		17	970
	2	974		18	970
•	3	972	. *	19	971
	4	973		20	969
	5	972	• • •	21	969
	6	973		22	667
•*	7	977		23	0
	8	971		24	265
	9	972		25	808
	10	970		26	968
	11	969	·· · · · ·	27	969
	12	970	÷	28	969
	13	547		29	969
	14	879		30	967
	15	968		31	968
	16	060	•		

INSTRUCTIONS:

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.	50-286
UNIT NAME	Indian Point.
DATE	01-04-88
TELEPHONE	914-739-8340

REPORT MONTH December 1987

No.	Date	Type	Duration (Hours)	Reason 2	Method of Shutting 3 Down Reactor	Licensee Event Report #	System Code	Component Code 5	Cause & Corrective Action to Prevent Recurrence
07	871213	F .	NA	В	NA	NA	CJ	PIPEXX A	In order to minimize personnel radiation exposure during maintenance on No. 34
									Reactor Coolant Pump, a load reduction from 100% reactor power to 35% reactor
					18	· · · ·			power was performed.
08	871222	F	34.17	В	3	87-012-00	IA	RelayX A	While performing a monthly surveillance test on the reactor coolant low flow
									protection system, a unit trip occurred. The manual tripping of a bistable trip
				•					switch, in conjunction with a high resistance contact on another relay,
								: :	established the low reactor coolant flow trip matrix. After the faulty relay was
									were inspected, the unit was returned to service.
			-		1 		s.:		
							•		

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F: Forced

S: Scheduled

Reason:

2

A- Equipment Failure (Explain)

B- Maintenance of Test

C- Refueling

D- Regulatory Restriction

E- Operator Training & License Examination

F- Administrative

G- Operational Error (Explain)

H- Other (Explain)

Method: 1- Manual

3

2- Manual Scram

3- Automatic Scram

4

5

4- Other (Explain)

Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

Exhibit H - Same Source

MONTHLY I & C CATEGORY I REPORT

				MONTH
WR#	DATE	EQUIPMENT	MALFUNCTION	CORRECTIVE ACTION
7698	12/8/87	Reactor Vessel Level Indicating System Thermocouples LO5, HO5, Pl3, Jll and Ll1.	Remote indication differs from computer indication.	Calibrated readings.
7707	12/8/87	Steam Generator Blowdown Radiation Monitor R-19.	Radiation monitor spiking.	Re-attached ground wire.
7659	12/11/87	Fan Cooler Unit Radiation Monitor R-16.	Erratic reading on recorder.	Replaced power supply and analog meter.
7759	12/11/87	Steam Generator Blowdown Radiation Monitor R-19.	Steam generator blowdown tank city water quench valve PCV-1227 did not automatically shut upon actuation of R-19 high alarm and "Loss of Counts" section could not be performed due to "electric noise".	Replaced solenoid for PCV-1227, and secured wire in cabinet of RM-80 that was inducing "noise".
7722	12/17/87	Rod Insertion Limit Computer Potentiometer.	High limit adjust potentiometer "open".	Replaced potentiometer.
7724	12/17/87	Power Range Nuclear Instrumentation Comparator and Rate Drawer.	Channel N41 channel deviation alarm drifted.	Adjusted isolation amplifier.
7811	12/22/87	Reactor Protection System, Reactor Coolant Loop 34 Low Flow Protection Relay FC-446X.	High resistance across contacts.	Replaced relay.
7607	12/24/87	Reactor Protection System, Reactor Coolant Average Teperature (Tave) Defeat Switch 3T-412A.	Loose switch contacts.	Replaced switch.

December 1987

MONTHLY MAINTENANCE CATEGORY I REPORT

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December 1987 MONTH

WR#	DATE	EQUIPMENT	MALFUNCTION	CORRECTIVE ACTION
12444	12/3/87	#32 Spent Fuel Pit Cooling Pump.	Pump seized.	Rebuilt pump.
12473	12/3/87	Reactor Protection System, Reactor Trip Bypass Breaker B.	Inoperable undervoltage relay.	Replaced undervoltage relay
10689	12/8/87	Fuel Storage Building Rolling Door.	Inoperable limit switch.	Replaced limit switch.
12390	12/9/87	#32 Charging Pump.	Discharge flange leak.	Replaced flange gasket.
12492	12/9/87	#32 Charging Pump Discharge Vent Valve.	Weld leak at valve connection.	Repaired weld.
12519	12/10/87	#35 Service Water Pump.	Corroded vacuum breaker plug.	Replaced plug.
10958	12/10/87	#33 Diesel Generator Pre Lube Pump On-Off Switch.	Switch does not function.	Repaired wiring to switch.
12525	12/13/87	#34 Reactor Coolant Pump, Component Cooling Water Isolation Valves 252D and 253D.	Packing leaks.	Tightened packing.
12107	12/14/87	#32 Service Water Pump.	Packing leak.	Repacked pump.
12447	12/15/87	#31 Charging Pump.	Packing leak.	Repacked pump.
12223	12/16/87	Station Air Make-up to Weld Channel Containment Pressurization Non-Regenative Dryer Inlet Isolation Valve IV-15-1.	Packing leak.	Adjusted packing.
12251	12/17/87	Electrical Heat Tracing, Circuit #38 Boric Acid Heat Tracing.	Open circuit.	Repaired broken splice.

MONTHLY MAINTENANCE CATEGORY I REPORT

December 1987 MONTH ·

WR#	DATE	EQUIPMENT	MALFUNCTION	CORRECTIVE ACTION
11979	12/22/87	#32 Instrument Air Compressor Unloader Solenoid.	Solenoid leaks-by.	Replaced solenoid valve.
12425	12/23/87	Main Feedwater System, #32 Steam Generator Feedwater Regulator Valve BFD-FCV-427.	Packing leak.	Replaced packing.
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SUMMARY OF OPERATING EXPERIENCE

DECEMBER 1987

Indian Point Unit No. 3 was synchronized to the bus for a total of 709.83 hours, producing a gross generation of 681,600 MWe.

On December 13, at approximately 0400 hours, while adding oil to No. 34 Reactor Coolant Pump's Lower Motor Bearing, a seal injection water leak was observed. In order to minimize personnel radiation exposure during maintenance on the pump, a load reduction from 100% reactor power to 35% reactor power was performed. After the leak was repaired, a load escalation to 100% reactor power was commenced on December 13, at 1545 hours. On December 14, at 1445 hours, the unit achieved 100% reactor power.

On December 22, after 108 days of continuous operation, a unit trip occurred at 1630 hours. The trip occurred while plant personnel were performing a monthly surveillance test on the Reactor Coolant Low Flow Protection System. It was determined that the tripping of a bistable trip switch associated with relay FC-444X, in conjunction with a high resistance contact on relay FC-446X, resulted in the de-energization of the Low Reactor Coolant flow matrix output relay, LF-4X, resulting in a reactor trip.

The relay was found to be faulty due to tarnish build-up. The relay was replaced with a new, identical relay and a sample consisting of all reactor coolant flow logic relays was developed and tested to verify that a similar condition did not exist elsewhere in the reactor protection racks.

The reactor was brought critical on December 23, at 2048 hours, and the unit was synchronized to the bus on December 24, at 0240 hours. The unit achieved 100% reactor power on December 25, at 1645 hours, and remained on line for the remainder of the reporting period.

Indian Point 3 Nuclear Power Plant P.O. Box 215 Buchanan, New York 10511 914 739.8200



January 14, 1988 IP3-WAJ-88-003Z IP3-LMK-88-006H

Docket No. 50-286 License No. DPR-64

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

Dear Sir:

Enclosed you will find the monthly operating report relating to Indian Point 3 Nuclear Power Plant for the month of December, 1987.

Very truly yours,

12 William A. Josiger

Resident Manager Indian Point 3 Nuclear Power Plant LK/sn/4:09

Enclosure

cc:

Mr. William Russell, Regional Administrator Region 1 U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, Pennsylvania 19406

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339