

OPERATING DATA REPORT

DOCKET NO. 50-286
 DATE 6-2-80
 COMPLETED BY C. Connell
 TELEPHONE (914) 739-8200

OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: May 1980
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 952
7. Maximum Dependable Capacity (Net MWe): 917
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	3647	32,904
12. Number Of Hours Reactor Was Critical	723.9	2346.3	23,931.1
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	698.1	1932.5	22,965.4
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,824,582	3,810,636	59,915,103
17. Gross Electrical Energy Generated (MWH)	526,070	1,044,700	19,409,801
18. Net Electrical Energy Generated (MWH)	503,021	981,862	18,625,299
19. Unit Service Factor	93.8	53.0	69.8
20. Unit Availability Factor	93.8	53.0	69.8
21. Unit Capacity Factor (Using MDC Net)	73.7	29.4	61.7
22. Unit Capacity Factor (Using DER Net)	70.1	27.9	58.7
23. Unit Forced Outage Rate	6.2	24.6	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Turbine Outage October

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
<u>N/A</u>	<u>N/A</u>
<u>N/A</u>	<u>N/A</u>
<u>N/A</u>	<u>N/A</u>

8006190 561

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286
Indian Point
UNIT No. 3
DATE 6-2-80
COMPLETED BY C. Connell
TELEPHONE 914-739-8200

MONTH May

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	576
2	573
3	576
4	578
5	576
6	614
7	757
8	756
9	753
10	758
11	825
12	846
13	848
14	847
15	845
16	632

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	335
19	282
20	406
21	849
22	846
23	842
24	849
25	847
26	825
27	823
28	821
29	822
30	557
31	496

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May

DOCKET NO. 50-286
 UNIT NAME Indian Point No. 3
 DATE 6-2-80
 COMPLETED BY C. Connell
 TELEPHONE (914) 739-8200

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
16	800516	F	15.5	A	3		EA	RELAYX-D	Misoperation of TR Relay at substation in Buchanan with multiple ground path between IP3 and substation. Ground associated with battery 31 cleared-up
17	800519	F	22.7	A	3		ED	GENERA-F	Voltage transient on 33 Instrument Bus Repaired components within 33 static inverter
18	800530	F	7.7	A	3		HA	INSTRU-E	False actuation of independent electric turbine overspeed protection system

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit H - Same Source

(9/77)

Summary of Operating Experience - May 1980

Indian Point Unit #3 was synchronized to the bus for a total of 698.1 hours producing a gross generation of 526,070 MWe for this reporting period.

During this period the unit experienced three plant trips.

On May 6th at 1330 commenced load escalation to 90% reactor power after reinstating #33 circulating water pump into operation. Maintained constant output until nuclear instrumentation and reactor protection system trips were readjusted as a result of the incore/excore nuclear instrumentation. On May 11th at 0315 commenced load escalation to 100% reactor power upon completion of Nuclear Instrumentation and Reactor Protection Circuit Calibrations.

On May 16th at 1810 the unit experienced an electrical trip attributed to the opening of Breaker 1 and 3 located at the substation in Buchanan. This was caused by misoperation of a TR relay caused by multiple ground paths between Buchanan substation and IP3. A ground associated with battery #31 at IP3 was located and cleared. On May 18th at 0941 the unit was returned to service and commenced load escalation to 100% reactor power.

On May 19th at 0841 the unit experienced a trip due to steam flow/feed flow mismatch coincident with low level in 32 steam generator. This was accompanied by a safety injection signal from high steam flow SIS logic. Both actuations were investigated and attributed to a voltage transient on 33 Instrument Bus. Further investigation revealed that the transient was initiated by electrical component failure within #33 static inverter. Instrument bus #33 was placed on the back-up feed while repairs were affected. On May 30th repairs and testing of #33 Static Inverter were completed and it was returned to service.

At 1619 on May 30th, the unit experienced a spurious turbine electrical overspeed trip during a rapid load reduction caused by #32 Heater Drain Pump tripping. Unit was returned to service at 2358 hours that same day and loaded consistent with heater drain pump capabilities.

MONTHLY MAINTENANCE REPORT

May 1980
Month

DATE	W.R.#	EQUIPMENT	MALFUNCTION	CORRECTIVE ACTION
5/27	I-1060	Steam Generator Blowdown Valves	Improper Valve Position	Faulty Limit Switch Replaced
5/25	I-1061	#31 Charging Pump	Block Cracked	Replaced Fluid Block
4/29	I-0844	FCV-1173	Improper Light Indication	Reset Limit Switch
4/24	I-1058	#34 Service Water Pump	Renew Pump Internals	Replaced Internals

MONTHLY I & C CATEGORY I REPORT

May 1980

Month

Date	W.R. #	Equipment	Malfunction	Corrective Action
NONE				

OPERATING DATA REPORT

MARCH

DOCKET NO. 50-286
DATE April 3, 1980
COMPLETED BY C. Connell
TELEPHONE (914) 739-8200

OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: March, 1980
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 952
7. Maximum Dependable Capacity (Net MWe): 917
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period			
12. Number Of Hours Reactor Was Critical			
13. Reactor Reserve Shutdown Hours			
14. Hours Generator On-Line			
15. Unit Reserve Shutdown Hours			
16. Gross Thermal Energy Generated (MWH)			
17. Gross Electrical Energy Generated (MWH)			
18. Net Electrical Energy Generated (MWH)		316,264	
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (Using MDC Net)		15.8	
22. Unit Capacity Factor (Using DER Net)		15.0	
23. Unit Forced Outage Rate			
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A
N/A
N/A

N/A
N/A
N/A

OPERATING DATA REPORT

DOCKET NO. 50-286
DATE 5-1-80
COMPLETED BY C. Connell
TELEPHONE (914) 739-8200

OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: April 1980
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 952
7. Maximum Dependable Capacity (Net MWe): 917
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period			
12. Number Of Hours Reactor Was Critical			
13. Reactor Reserve Shutdown Hours			
14. Hours Generator On-Line			
15. Unit Reserve Shutdown Hours			
16. Gross Thermal Energy Generated (MWH)			
17. Gross Electrical Energy Generated (MWH)			
18. Net Electrical Energy Generated (MWH)		478,841	
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (Using MDC Net)		18.0	
22. Unit Capacity Factor (Using DER Net)		17.1	
23. Unit Forced Outage Rate			
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A
N/A
N/A

N/A
N/A
N/A

REGULATORY DOCKET FILE COPY

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Docket Files 50-247/286

CSParrish

ORB1 Reading

LN01shan

June 5, 1980

Docket No. 50-247/286

Director, Technical Development Programs
State of New York Energy Office
Agency Building 2, Empire State Plaza
Albany, New York 12223

Subject: CONSOLIDATED EDISON COMPANY OF NEW YORK (Indian Point 2)
POWER AUTHORITY OF THE STATE OF NEW YORK (Indian Point 3)

The following documents concerning our review of the subject facility
are transmitted for your information:

- ☐ Notice of Receipt of Application.
- ☐ Draft/Final Environmental Statement, dated _____
- ☐ Safety Evaluation, or Supplement No. _____, dated _____
- ☐ Notice of Hearing on Application for Construction Permit.
- ☐ Notice of Consideration of Issuance of Facility Operating License.
- ☐ Application and Safety Analysis Report, Vol. _____
- ☐ Amendment No. _____ to Application/SAR, dated _____
- ☐ Construction Permit No. CPPR- _____, dated _____
- ☐ Facility Operating License No. DPR- _____, dated _____
- ☐ Amendment No. _____ to CPPR- _____ or DRR- _____, dated _____
- ☒ Other: May 30, 1980 Order _____
- ☐ _____

Division of Licensing and Regulation

Enclosures:
As stated

cc: The Honorable George Begany
Mayor, Village of Buchanan

18 168 Westchester Avenue
Buchanan, New York 10511

OFFICE

SURNAME

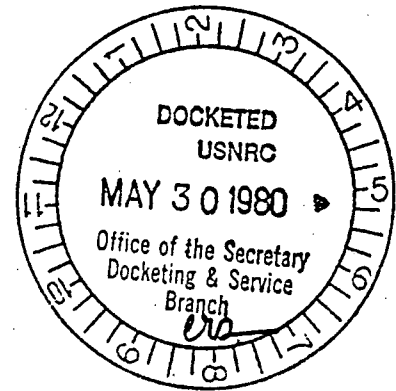
DATE

DL:ORB1 *cp*
CSParrish:jb
06/5/80

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

COMMISSIONERS:

John F. Ahearne, Chairman
Victor Gilinsky
Richard T. Kennedy
Joseph M. Hendrie
Peter A. Bradford



In the Matter of

CONSOLIDATED EDISON COMPANY OF
NEW YORK, INC. (Indian Point, Unit
No. 2)

POWER AUTHORITY OF THE STATE OF NEW
YORK (Indian Point, Unit No. 3)

Docket Nos. 50-247
50-286

ORDER

On February 11, 1980, the Director of the Office of Nuclear Reactor Regulation issued a decision granting in part and denying in part a petition, filed by the Union of Concerned Scientists ("UCS"), that called for, among other things, the decommissioning of Indian Point Station Unit 1, and the shutdown of Units 2 and 3.

On February 15, 1980, the Commission approved publication of a notice soliciting public comment on the Director's decision regarding Units 2 and 3. 45 Fed. Reg. 11969 (Feb. 22, 1980). The notice requested comment both on the merits of the Director's decision and on the procedural form which any further Commission action on the matter should take. The notice included a list of five possible procedural options, including adjudication, rulemaking, and an informal proceeding. It noted that this list was not exhaustive, nor were the options necessarily mutually exclusive.

The Commission received well over 100 responses to this Federal Register notice, and has given them careful consideration. Our review of the original UCS petition, the Director's February 11 decision, and the comments which have been received lead us to a four-pronged approach for resolving the issues raised by the UCS petition. These are described in more detail below but, briefly, the four prongs are these: (A) as an exercise of Commission discretion, an adjudication before an Atomic Safety and Licensing Board, with the decision on the merits made by the Commission itself, on safety issues related specifically to Indian Point Units 2 and 3; (B) an informal proceeding, to begin at once, for the purpose of determining, on an expedited basis, the issues which the adjudicatory proceeding is to address, and the criteria to be used for the ultimate decision in that proceeding; (C) generic consideration of the question of operation of reactors in areas of high population density; and (D) the establishment of an intra-agency task force to review available data, including the comments already received, and present the Commission with all relevant information so that the Commission itself can decide the status of the Indian Point 2 and 3 reactors during the pendency of the adjudication (i.e., in operation, shut down, or in operation under further restrictions).

Since one purpose of the informal proceeding which we initiate today is to help shape the discretionary hearing which will follow it, the Commission of course cannot today define with precision the specifics of that adjudication. Moreover, in the event that the informal proceeding results in an order to suspend, revoke, or modify the licensees' operating licenses, the licensees will have a right to an adjudicatory hearing, pursuant to section 189a. of the Atomic Energy Act. Such a hearing could obviate, in large part, the discretionary hearing contemplated by this Order. Despite this necessary uncertainty, we

believe it would serve the interests of all concerned for the Commission to indicate its present intent with respect to the discretionary adjudication which, unless our course of action results in an adjudication as of right under section 189a., we intend to institute once the informal proceeding is concluded.

A. The Planned Adjudicatory Proceeding

The Commission's present intent is that the discretionary proceeding will be conducted in the vicinity of Indian Point by an Atomic Safety and Licensing Board, using the full procedural format of a trial-type adjudication, including discovery and cross-examination. The purpose of the proceeding will be to take evidence and make recommended findings and conclusions on disputed issues material to the question whether the Indian Point Units 2 and 3 plants should be shut down or other action taken. The record of the proceeding, together with recommendations, will then be forwarded to the Commission for the final agency decision on the merits of the proceeding. Specifically, the Licensing Board will be asked to address a series of questions related to the safety of operation of Indian Point Units 2 and 3. Subject to modification in light of the informal proceeding, those questions will be as follows:

1. What is the current status and acceptability of state and local emergency planning within a 10-mile radius of the site and, to the extent that it is relevant to risks posed by the two plants, beyond a 10-mile radius?
2. What improvements in the level of emergency planning can be expected in the near future, and on what time schedule?

3. What improvements in the level of safety will result from measures required or referenced in the Director's Order to the licensees, dated February 11, 1980?—*/
4. What risk (probability and consequences) may be posed by serious accidents at Indian Point 2 and 3, including accidents not considered in the plants' design basis, pending and after any improvements described in (2) and (3) above?
5. Based on the foregoing, how do the risks posed by Indian Point Units 2 and 3 compare with the range of risks posed by other nuclear power plants licensed to operate by the Commission? (The Board should limit its inquiry to generic examination of the range of risks and not go into any site-specific examination other than for Indian Point itself.)
6. What would be the energy, environmental, economic or other consequences of a shutdown of Indian Point Unit 2 and/or Unit 3?

B. The Informal Proceeding

By this Order, the Commission institutes an informal proceeding for the purpose of better defining the adjudicatory proceeding that will follow it. To this end, the Commission solicits the views of interested members of the public on the following questions:

1. To what extent are answers to the questions listed in Section (A) above material or useful in resolving the ultimate issue in the adjudication -- i.e., operation, shutdown, or modification of the Indian Point 2 and 3 plants?

 */ A contention by a party that one or more specific safety measures, in addition to those identified or referenced by the Director, should be required as a condition of operating the facility or facilities, would be within the scope of this inquiry.

2. By what criteria should the acceptability of the risk posed by these facilities be determined?

The Commission recognizes that on both these questions, guidance from the Commission to the Atomic Safety and Licensing Board conducting the adjudication is desirable. The Commission requests that, in the interest of an expeditious determination on these matters of high public importance, comments be filed (i.e., placed in the U.S. mails) within 21 days of the issuance of this Order. The Commission recognizes that for a significant number of those who filed comments in response to the February 15 solicitation of public views, their filings in this informal proceeding will duplicate to some extent the comments filed in response to the February 15 notice. Accordingly, all persons filing in this informal proceeding are at liberty to incorporate by reference any comments they may have filed at an earlier time.

C. The Generic Proceeding

The Commission will make its determination as to the form for generic consideration of the question of operation of reactors in areas of high population density after it has concluded the informal proceeding.

D. The Task Force on Interim Operation

The Commission has determined that more information is necessary in order to make a decision as to the status of the two facilities (i.e., operating, shut down, operating at reduced power, or otherwise) during the pendency of the adjudication contemplated in this Order. Accordingly,

the Commission hereby directs the General Counsel and the Director, Office of Policy Evaluation, to establish a task force to prepare a report to the Commission on the information available at this time that bears on our decision whether to permit interim operation of Indian Point Units 2 and 3. The Commission expects that the Task Force will include persons from the Office of Nuclear Regulatory Research. It is the Commission's expectation that the Task Force will supply information necessary for a judgment on interim operation, but will itself make no recommendation on that question.

The Task Force Report should include information on at least the following:

1. A description of the Indian Point site demography as compared to other U.S. reactor sites.
2. A comparison of reactor accident risks (spectrum of probabilities and consequences for health impacts and property damage) at the Indian Point site to reactor accident risks at other sites.
3. The effects of potential public emergency response systems (evacuation, sheltering, etc.) on reactor accident risks at Indian Point. The area studied should be large enough to include New York City. This evaluation should include an assessment of the effects of uncertainties associated with successfully completing such actions.
4. A comparison of the reliability or accident probabilities of the Indian Point 2 and 3 reactors to each other and to other reactor designs which have been analyzed. This should include consideration of the changes ordered by the Director, NRR; technical design comments received in response to the

Commission's February 15 solicitation of comments; and the effects of partial reductions in power level.

5. An assessment of the economic, social, and other "non-safety" effects of shutting down or reducing the output of either or both reactors.

The General Counsel and the Director, OPE, should submit this report to the Commission by June 12, 1980.

Commissioner Bradford's separate views are attached.

It is so ORDERED.*

For the Commission



SAMUEL J. CHILK
Secretary of the Commission

Dated at Washington, D.C.
this 30th day of May, 1980.

*Section 201 of the Energy Reorganization Act, 42 U.S.C.s5841 provides that action of the Commission shall be determined by a "majority vote of the members present." Commissioner Gilinsky was not present at this Affirmation Session. Had Commissioner Gilinsky been present at the meeting he would have voted to approve the issuance of the Order for the proceedings, but would have preferred his own format of the Order, which contained several points that he thought should have been included in the Order being affirmed today. Accordingly, the formal vote of the Commission was 4-0 in favor of the decision.

SEPARATE VIEWS OF COMMISSIONER BRADFORD:

The Commission has never required reviews for operating reactors to show the extent to which they meet current NRC Regulatory Guides and Branch Technical Positions. Consequently, for Indian Point Units 2 and 3, we simply do not know whether or how these plants meet current standards. The differences between our older and current standards can be very significant, as exemplified by environmental qualification standards. If the Commission is to make a well-informed judgment about the risks posed by the operation of these plants, then I believe it is imperative that the Licensing Board receive evidence on the issue of the extent to which Indian Point Units 2 and 3 meet current Regulatory Guides and Branch Technical Positions. Additionally, the Board should receive evidence on what unresolved safety questions are applicable to Units 2 and 3. Under the terms of the footnote on page 4, the Board may receive evidence on these issues if a party raises them as contentions. Depending solely on intervenors to raise whatever Regulatory Guides interest them is no substitute for a systematic analysis by the Commission staff.