Indian Point 3 Nuclear Power Plant P.O. Box 215 Buchanan, New York 10511

914 736.8001



John H. Garrity Resident Manager

May 28, 1993 IPN-93-043

Docket No. 50-286 License No. DPR-64

Document Control Desk Mail Station PI-137 U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Dear Sir:

The attached revised Licensee Event Report LER 93-004-01 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in the requirements per 10CFR50.73 (a)(2)(i)(B). No new commitments are being made by this submittal.

Very truly yours,

John H. Garrity' Resident Manager Indian Point Three Nuclear Power Plant

JHG/fp

cc: Mr. Thomas T. Martin Regional Administrator Region 1 U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406

> INPO Records Center 700 Galleria Parkway Atlanta, Georgia 30339-5957

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RC FORM 366 5-92)		U.S. NUCLEAR REGULATORY COMMISSION						APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95					
LICENSEE EVENT REPORT (LER)							ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT WASHINGTON DC 20503						
FACILITY NAME (1)								DOCKET NUMBER (2) PAGE					PAGE (3)
Indian Point	Unit	3								05000286	5		LOF 4
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NRC FORM 366A (5-92)	U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95					
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION				ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND RUDGET WASHINGTON DC 20503.					
FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)	>	PAGE (3	5)		
Indian Point Unit 3	Point Unit 3	05000286	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER				
			93	04	01	2 OF	4		

DESCRIPTION OF THE EVENT

At 1310 on December 29, 1992, Operations department personnel declared No.32 ABFP inoperable because it failed surveillance test 3PT-M2OA, "Surveillance and Inservice Test Auxiliary Boiler Feed Pump Functional Test." Instumentation and Controls (I&C) Engineering personnel determined that the overspeed trip microswitch had failed because of over travel of the trip arm. This stressed the reset (flex) capability of the switch so that it would not remain reset when the reset push-button was released.

I&C technicians replaced the microswitch (Micro Switch Model No. 1 BZE6-RN2X1) and adjusted the travel of the trip arm.

INVESTIGATION OF THE EVENT

On October 30, 1992 the trip/reset microswitch on No. 32 ABFP was replaced. The original switch had failed due to age related fatigue. After the replacement, the I&C technicians attempted to adjust the switch even though they had no experience with this type of switch and found no relevant adjustment guidance in the vendor literature. I&C personnel should have instituted a change of scope to the work request which should have resulted in development of detailed written switch adjustment instructions. On October 31, 1992 the overspeed trip mechanism was satisfactorily retested. In November and December 1992, No. 32 ABFP was satisfactorily tested using the routine monthly surveillance test. On December 29 the monthly surveillance test was performed and No. 32 ABFP failed. Investigation revealed that the microswitch that was replaced two months earlier had failed.

An evaluation of the microswitch revealed that the failure mechanism was such that it could only have occurred during contact of the trip arm with the microswitch. The last time this occurred was when the trip was actuated at the completion of the last test (December 3, 1992). The failure of the microswitch remained undetected because the surveillance test procedures did not verify that the microswitch had reset.

NRC FOR (5-92)	M 366A U.S. NUCLEAR RE	APPROVED BY ONB NO. 3150-0104 EXPIRES 5/31/95						
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	FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)		
	Indian Point Unit 3	05000286	year 93	SEQUENTIAL NUMBER	REVISION NUMBER 01	3 OF 4		
	CAUSE	OF THE EVEN	T	····				
	A cause of the event was pers improperly adjusted the micro cause was that the surveillan M20B, were inadequate. These left condition of No.32 ABFP	onnel error switch on Oc ce test proc procedures was ready fo	in ju tober edure did r r aut	ndgement. 30, 1992 es, 3PT-M2 not ensure comatic op	I&C po . A so OA and that f eration	ersonnel econd 3PT- the as n.		
	CORRE	CTIVE ACTION	6					
	Corrective actions to prevent recurrence of this type of event are as follows:							
	All I&C and Maintenance department personnel who are qualified to perform this type of maintenance have been trained on this LER. This training emphasized attention to detail especially in maintenance activities which involve instruments, limit switches and devices which require adjustments;							
	The I&C department has revised Administrative Directive IC-AD-8, "Work Processing," to require that I&C technicians consult with their supervisor for additional direction when troubleshooting identifies the need to replace equipment and devices;							
	The procedure inadequacy has been addressed by revising the surveillance procedures, 3PT-M2OA and B, "Surveillance and Inservice Test Auxiliary Boiler Feed Pump Functional Test," so that they verify automatic starting capability has been restored to No.32 ABFP at the completion of the functional test.							
	ANALYSIS OF THE EVENT							
	This event is reportable under licensee shall report any oper plant's technical specification 3.4.C.1 states "With one auxi the pump to operable status w within the next 12 hours." The states "With two auxiliary feas shutdown within 12 hours." But the required compensatory act	r section 10 ration or co ons. Techni liary feedwa ithin 72 hou echnical spe edwater pump ecause the c ions were no	CFR nditi cal s ter p rs or cific s inc ondit t tak	50.73(a)(on prohib specificat oump inope be in ho cation sec operable, ion remain ten.	2)(i)(I ited by ion sec rable, t shutc tion 3, be in B ned unc	B). The the tion restore lown 4.C.2 not letected		

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	FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)						
	Indian Point Unit 3	05000286	YEARSEQUENTIAL NUMBERREVISION NUMBER93 00401						
	SAFETY	SIGNIFICANC	::::::::::::::::::::::::::::::::::::::	ק					
	This event had no effect on t public. One ABFP was always function required in the Fina time a second ABFP was consid inoperable or its emergency p was 71 hours and 22 minutes.	he health an operable to l Safety Ana ered inopera ower source	d safety of the general perform the heat removal lysis Report. The cumulative ble (either the pump inoperable due to testing)	11					
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	SECURING	FROM THE EV	ENT						
	The overspeed trip microswitch was replaced, proper adjustments were made, and the appropriate surveillance tests were revised. The No.32 ABFP was declared operable at 2030 hours, on December 29, 1992.								
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