

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914-736-8000



**New York Power
Authority**

January 21, 1993
IP3-NRC-93-009

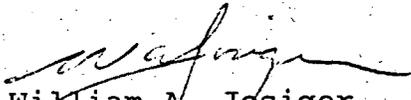
Docket No. 50-286
License No. DPR-64

Document Control Desk
Mail Station PI-137
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Sir:

The attached Licensee Event Report LER 93-003-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in the requirements per 10CFR50.73(a)(2)(i)(B).

Very truly yours,


William A. Josiger
Resident Manager
Indian Point Three Nuclear Power Plant

waj/da/rj
Attachment

cc: Mr. Thomas T. Martin
Regional Administrator
Region 1
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

260042

9301280175 930121
PDR ADDCK 05000286
S PDR

IF22
11

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	PAGE (3) 1 OF 0 4
---	---	-----------------------------

TITLE (4)
Missed Surveillance on Gaseous Effluent Flow Rate Monitor

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)					
1	2	0	4	9	2	9	3	0				0	5	0	0	0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)										
POWER LEVEL (10) 1 1 0 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)				
	20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)				
	20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
	20.405(a)(1)(iii)		X 50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
	20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)

NAME Doug Ames, Licensing Coordinator	TELEPHONE NUMBER
	AREA CODE: 9 1 4 NUMBER: 7 3 1 6 1 8 1 0 2 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On December 4, 1992 Indian Point 3 was at full power. A system engineer identified that the Radioactive Machine Shop building exhaust ventilation flow rate monitor had exceeded the refueling calibration technical specification frequency. An investigation determined the monitor was used in release calculations from October 1, 1991 to December 4, 1992 without a current refueling calibration. The causes were lack of effective communication between a project engineer and the Performance and Reliability Group and the failure of an I&C staff member to enter the test into the I&C internal test scheduling program. The staff members involved with the lack of effective communication and the I&C staff member have been apprised of this LER to ensure an understanding of the implications of the event and the need for effective communications, and attention to detail. The monitor was returned to service on December 23, 1992.

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 3	- 0 0 3	- 0 1 0	0 2	OF	0 4

EXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF THE EVENT

On December 4, 1992 with the unit at full power, the radiation monitor system engineer identified that the Radioactive Machine Shop (RAM) building exhaust ventilation flow rate monitor (FI) had missed its refueling frequency calibration. At 1315 on December 4, 1992 the monitor was declared inoperable and compensatory action was taken as per Technical Specification, including taking the RAMS Building exhaust out of service.

The RAMS building ventilation flow rate monitor consists of eight flow sensors mounted in the building vent. The signals from the sensors are summed and then sent as an indication signal to the RAMS building effluent gaseous activity monitor (R-59). The device was used in gaseous release calculations starting October 1, 1991.

The technical specification calibration requirements for the flow monitor are a daily channel check, quarterly functional test, and a channel calibration at a refueling frequency. The requirements are described in IP3 technical specification appendix B (RETS) table 3.2-1.

The eight sensors of the flow element were removed and sent to the manufacturer for recalibration during the 1989 refueling outage under the work control system. The 1990 refueling test of the flow element was not performed due to an ineffective communication between a project engineer and the Performance and Reliability Group. The Performance and Reliability staff wrongly believed that the test was performed in 1990 by the flow monitor vendor, similar to the 1989 test that was arranged by the project engineer. Therefore, the surveillance program reflected a 1992 refueling test as the next required refueling test to be performed. The deadline for completion was October 10, 1992.

The test exceeded the October 10, 1992 deadline due to the failure of an I&C staff member to enter the test into the I&C internal test scheduling system. Contributing causes were the lack of an effective escalation process for management to resolve delays in the conduct of surveillance tests and delays in providing test support equipment.

The monitor was returned to service on December 23, 1992.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 3	- 0 1 0 3	- 0 0 0	3	OF 0 4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CAUSE OF THE EVENT

The cause for using the flow rate monitor device for release calculations since October 1, 1991 without having met calibration requirements was the lack of effective communication between a project engineer and the Performance and Reliability Group.

A contributing cause to using the monitor in release calculations without current calibration was the lack of formal controls to ensure Offsite Dose Calculations Manual changes requiring surveillance program updates are completed prior to implementation.

The cause for the test exceeding the October 10, 1992 deadline was due to the failure of an I&C staff member to enter the test into the I&C internal test scheduling system.

CORRECTIVE ACTIONS

The following corrective actions are planned or are completed to prevent recurrence of the event:

1. The project engineer and Performance staff members involved with the lack of effective communication have been apprised of this LER to ensure an understanding of the implications of the event and the need for effective communication and attention to detail.
2. The I&C staff member involved with the inappropriate action of not entering the test into the I&C internal test scheduling system has been apprised of this LER to ensure an understanding of the implications of the event and the need for attention to detail.
3. Formal controls will be established by March 31, 1993 to ensure revisions to the IP3 Offsite Dose Calculation Manual (ODCM) requiring surveillance program updates are completed prior to implementation.
4. On December 9, 1992 the Technical Services Manager instituted a weekly report of surveillance tests not performed by an early completion date to department managers and general managers. This will escalate missed surveillances to management's attention for resolution prior to the end of the technical specification frequency time.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0500028693	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		93	003	00	04	OF 04

TEXT (If more space is required, use additional NRC Form 388A's) (17)

- On December 16, 1992 the RAMS Building Vent Flow Rate Monitor Calibration Check on the flow element was completed. The results of the tests revealed that the flow elements were in calibration. The monitor was returned to service on December 23, 1992.
- On August 3, 1992 a system engineer was assigned to the Radiation Monitor system area including the RAMS building flow rate monitor. The effectiveness of this assignment was demonstrated by the fact that the system engineer was the one who made initial discovery of this event.

ANALYSIS OF THE EVENT

The event is reportable under 10CFR50.73(a)(2)(i)(B). In accordance with plant technical specifications appendix B, section 3.2, table 3.2-1, item 4d., the flow rate measurement device for the RAMS vent flow rate monitor is required to be tested. The monitor was used in release calculations without a current refueling calibration from October 1, 1991 to December 4, 1992.

SAFETY SIGNIFICANCE OF THE EVENT

This event had no impact on the health and safety of the public. Based on the flow element being found in calibration on December 16, 1992, the missed surveillance calibrations did not impact the offsite dose calculations.

During normal operations greater than 99 percent of all gaseous releases occur through the main plant vent with negligible amounts released from the radioactive machine shop vent.

SECURING FROM THE EVENT

The monitor was returned to service on December 23, 1992.