

Indian Point 3  
Nuclear Power Plant  
P.O. Box 215  
Buchanan, New York 10511  
914 739.8200



August 14, 1991  
IP3-91-047

Docket No. 50-286  
License No. DPR-64

Document Control Desk  
Mail Station PI-137  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

The attached Licensee Event Report LER 91-010-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in the requirements per 10CFR50.73(a)(2)(i)(B).

Very truly yours,

A handwritten signature in cursive script, appearing to read 'Joseph Russell'.

Joseph Russell  
Resident Manager  
Indian Point Three Nuclear Power Plant

JM/rj  
Attachment

cc: Mr. Thomas T. Martin  
Regional Administrator  
Region 1  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	PAGE (3) 1 OF 0 3
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TITLE (4)  
Missed Special Report for Out of Service Steam Line Radiation Monitor

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	1 8	9 1	9 1	0 1 0	0 0 0	0 8	1 4	9 1			0 5 0 0 0

OPERATING MODE (9)  N THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

POWER LEVEL (10) 0 9 0	20.402(b)	20.406(c)	50.73(e)(2)(iv)	73.71(b)
	20.406(a)(1)(i)	50.38(c)(1)	50.73(e)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.38(c)(2)	50.73(e)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i) <input checked="" type="checkbox"/>	50.73(e)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(e)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(e)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Joseph Macchiarulo, OERG Engineer	TELEPHONE NUMBER
	AREA CODE: 9 1 4      7 3 6 8 0 4 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
A	I L R T		G 0 6 3	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)       NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During an assessment of the Indian Point 3 Radiation Monitoring System, the Power Authority discovered that 32 steam generator steam line radiation monitor, R-62A, had been out of service beyond seven days and a special report was not submitted to the NRC as required by technical specifications. This event was caused by a failure to recognize the reporting requirements associated with this technical specification. As a corrective action, the site licensing group will provide an independent review of all plant occurrence reports for reportability.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 1	0 1 0	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF THE EVENT

On July 18, 1991, during a review of out-of-service data on radiation monitors, the Power Authority recognized that the number 32 steam generator steam line radiation monitor, R-62A (Sorrento model E-115-1366) (G063) (IL) (RT), had been out of service beyond seven days without a special report being submitted within fourteen days of the event as required by plant technical specifications.

INVESTIGATION OF THE EVENT

On January 6, 1991, intermittent spiking was observed on the number 32 steam generator steam line radiation monitor, R-62A. The air ejector exhaust activity monitor (R-15) and the steam generator blowdown activity monitor (R-19) were verified normal.

R-62A was declared out of service and a work request was issued for repairs. Plant personnel overlooked a technical specification 14-day reporting requirement.

Trouble-shooting efforts revealed that the intermittent spiking was the result of a faulty detector.

The detector was replaced, and R-62A was retested satisfactorily. R-62A was returned to service on March 5, 1991.

CAUSE OF THE EVENT

This event was caused by a failure to recognize the 14-day reporting requirements associated with this technical specification.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 1	0 1 0	0 0	0 3	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTIONS

Personnel involved in this event have been advised of the need to accurately identify plant technical specification reporting requirements when placing equipment out of service. Additionally, the site licensing group will provide an independent review of all plant occurrence reports for reportability.

ANALYSIS OF THE EVENT

This event is reportable under 10CFR50.73 (a) (2) (i) (B) because R-62A was out of service beyond seven days and a special report was not submitted to the NRC as required by technical specifications. (The special report is required within fourteen days of the monitor being placed out of service if the monitor is out of service for more than seven days.)

The significance of this event is mitigated by the following factors:

- 1) any activity present in #32 steam generator would have been detected by R-15 (the air ejector exhaust activity monitor) and R-19 (the steam generator blowdown activity monitor) which were both in service while R-62A was inoperable
- 2) the Indian Point 3 chemistry department has the capability of sampling the steam header for number 32 steam generator during accident conditions in accordance with plant procedures.

SECURING FROM THE EVENT

The intermittent problem with R-62A was determined to be caused by a faulty detector. The detector was replaced and R-62A was successfully retested and declared operable on March 5, 1991.