

Indian Point 3
Nuclear Power Plant
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Joseph E. Russell
Resident Manager

April 5, 1991
IP3-91-013

License No. 50-286
Docket No. DPR-64

Charles E. Rossi, Director
Division of Operational Events Assessment
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
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Subject: Indian Point 3 Nuclear Power Plant
NRC Bulletin No. 89-02, Stress Corrosion Cracking
of High-Hardness Type 410 Stainless Steel Internal
Preloaded Bolting in Anchor Darling Model S350W
Swing Check Valves or Valves of Similar Design

References: 1. Letter IP3-90-011, dated March 14, 1990
from Joseph E. Russell to Charles E.
Rossi, "Authority Interim Response to
NRC Bulletin No. 89-02".

Dear Mr. Rossi:

This letter provides the Authority's second response to NRC
Bulletin No. 89-02. In Reference (1) the Authority provided
its interim response and schedule for completion of the
Bulletin actions.

The Authority has completed its documentation review of
similarly designed safety related check valves (with ASTM A-
193 Grade B6 Type 410SS block bolts), as requested in the
Bulletin.

That review identified thirty three (33) check valves that
fit the Bulletin criteria at Indian Point 3 (IP3). Thirteen
(13) of those valves are in primary systems, susceptible to
stress corrosion cracking.

Of the thirteen (13) primary system valves, five (5) were scheduled for disassembly and inspection during the Cycle 7/8 Refueling Outage. The remaining eight (8) primary system valves are identical in service or function to the five selected valves and were not initially selected for inspection.

The inspection results, for the five valves, showed one valve with minor cracking on both retaining block bolts. A hardness test performed on those bolts indicated a Rockwell hardness value greater than Rc 26. The bolts were replaced with ones that meet the Bulletin hardness criteria.

The other four valves' retaining block bolts showed no cracking and were in satisfactory condition. As a conservative measure, they were replaced with bolts that meet the Bulletin hardness criteria.

The inspection was then expanded, as a result of the minor cracking found on the one (1) valve, to include three (3) additional valves from the remaining eight (8) primary system valves. Their bolts were also replaced with ones that meet the Bulletin hardness criteria, even though no bolt degradation was found.

The remaining twenty (20) similarly designed safety related check valves are in secondary systems of low susceptibility to stress corrosion cracking. Four (4) of those twenty valves were inspected during the Cycle 7/8 Outage under the IP3 check valve preventative maintenance program. No stress corrosion cracking was found.

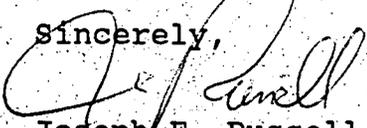
Inspection of the remaining five (5) primary and sixteen (16) secondary system valves will occur under the IP3 check valve preventative maintenance program. The Authority believes this is a technically justified approach for inspection of those twenty one (21) check valves. This approach is supported by the inspection results for the eight primary valves, the maintenance history for the remaining valves, and the location of the secondary system valves in systems of low susceptibility to stress corrosion cracking.

The Authority's check valve preventative maintenance program sets valve inspection frequency based on service condition, design parameters, and maintenance history for each valve.

The Authority will maintain review and inspection documentation and complete all reporting requirements as requested in the Bulletin.

Should you or your staff have any questions regarding this matter, please contact Mr. M. Peckham of my staff.

Sincerely,



Joseph E. Russell
Resident Manager
Indian Point Unit 3
Nuclear Power Plant

JER:RAL/rl

cc: Mr. Thomas T. Martin
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