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John C. Brons
Executive Vice President
Nuclear Generation

October 24, 1990
IPN-90-053

U.S. Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Revised Response to Generic Letter 90-04

- References:
1. Generic Letter 90-04, dated April 25, 1990, "Request for Information on the Status of Licensee Implementation of Generic Safety Issues Resolved with Imposition of Requirements or Corrective Actions"
 2. Letter from J. C. Brons, dated July 3, 1990, "Status of Implementation of Generic Safety Issues"

Dear Sir:

In accordance with Generic Letter 90-04 (Reference 1), the Authority provided the status on Generic Safety Issues (Reference 2). This letter inadvertently omitted the status of A-35, "Adequacy of Offsite Power Systems," for the Indian Point 3 Nuclear Power Plant. In addition, the status of GSI 51, "Improving the Reliability of Open-Cycle Service Water Systems," is being revised to appropriately reflect its incomplete status. The revised pages are enclosed.

If you should have any questions or concerns, please contact Mr. P. Kokolakis of my staff.

Very truly yours,

A handwritten signature in cursive script that reads 'John C. Brons'.
John C. Brons
Executive Vice President
Nuclear Generation

cc: see next page

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cc: U. S. Nuclear Regulatory Commission
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Resident Inspector's Office
Indian Point 3
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Mr. J. D. Neighbors, Sr. Project Manager
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Division of Reactor Projects-I/II
U.S. Nuclear Regulatory Commission
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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS**</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	IPN-84-020 8-13-84
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & MMP-1	N/A	IPN-85-003 1-15-85 IPN-84-049 10-31-84
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C	IPN-84-035 8-31-84 IPN-82-032 4-13-82
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	N/A	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	IPN-80-031 3-13-8

FACILITY NAME: Indian Point 3 Nuclear Power Plant
 DOCKET NO.: 50-286
 LICENSEE: New York Power Authority

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS**</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	N/A	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	N/A	
43 (B107)	Reliability Of Air Systems	All Plants	I	See Note 1
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	See Note 3
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	IPN-84-020 6-29-84
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	IPN-86-005 1-7-86 IPN-83-092 11-7-83 IPN-85-027 6-5-85
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	IPN-83-092 11-7-83

* Status codes are defined by the NRC Attachment to Enclosure 1 to Generic Letter 90-04 as follows: C - Completed, date of completion; I - Incomplete, expected date of completion; NA - Not applicable; E - Response to Generic Letter not completed, expected response date; NC - No changes were necessary.

** The letters listed in the comments column identify correspondence from the Power Authority to the NRC. These letters do not certify completion, but provide detailed information which in most cases was the basis for an associated NRC Safety Evaluation Report.

Notes:

1. In letter IPN-89-012, dated February 17, 1989, the Authority provided a 180-day response to Generic Letter 88-14, "Instrument Air Supply System Problems Affecting Safety-Related Equipment." This letter stated that the Authority plans to complete the requested verification, including any testing determined necessary, prior to startup from the Cycle 7/8 refueling outage.
2. In letters IPN-89-001 and IPN-89-008, dated January 3, 1989 and February 7, 1989, respectively, the Authority responded to Generic Letter 88-17, "Loss of Decay Heat Removal." IPN-89-008 stated that programmed enhancements requiring hardware changes or significant plant testing will be implemented prior to start-up following the Cycle 7/8 refueling outage.
3. The Authority's response to NRC Generic Letter 89-13, "Service Water System Problems Affecting Safety-Related Equipment, " was completed and forwarded to the NRC by letter IPN-90-004, dated February 9, 1990. The Authority plans to complete this issue by the end of 1991.