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John C. Brons
Executive Vice President
Nuclear Generation

November 17, 1989
IPN-89-070

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Response to Generic Letter 89-21
Implementation Status of Unresolved Safety Issue (USI) Requirements

Reference: Generic Letter 89-21: "Request for Information Concerning Status of Implementation of Unresolved Safety Issue (USI) Requirements, dated October 19, 1989.

Dear Sir:

The referenced Generic Letter requested that the Authority provide information concerning Indian Point 3 (IP3) specific implementation of NRC unresolved safety issues (USI). Attachment I to this letter provides the closure status of each USI identified in Generic Letter 89-21.

Should you or your staff have any questions regarding this matter, please contact Mr. P. Kokolakis of my staff.

Very truly yours,

For

John C. Brons
Executive Vice President
Nuclear Generation

cc: see next page

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ATTACHMENT I TO IPN-89-070

IMPLEMENTATION STATUS OF USI REQUIREMENTS FOR INDIAN POINT 3

New York Power Authority
Indian Point 3 Nuclear Power Plant
Docket No. 50-286
DPR-64

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	C 11/4/86	Note 1.
A-2/ MPA D-10 (TAC 68773)	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C 3/10/86	Note 2.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	C 6/18/86	Note 3.
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	NA	
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	

* C - COMPLETE
NC - NO CHANGES NECESSARY
NA - NOT APPLICABLE
I - INCOMPLETE
E - EVALUATING ACTIONS REQUIRED

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA	
A-9 (TAC 59104)	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	C 6/89	Note 4.
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	NA	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	C 11/12/82	Note 5.
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWR	C 12/2/83	Note 6.
A-17 (TAC 51099)	Systems Interactions	Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	E 9/1/92	Note 7.
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C 6/5/85	Note 8.

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-26/ MPA B-04 (TAC 59257)	Reactor Vessel Pressure Transient Protection	DOR Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	E 10/90	Note 9.
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	NA	
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C Phase I: 2/13/85 Phase II: 6/28/85	Note 10.
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	NA	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	I	Note 11.
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	C 12/3/85	Note 12.
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	I	Note 13.
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	E 9/1/92	Note 14.
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	I	Note 15.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	E 3/19/90	Note 16.
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	NA	
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C 1/23/86	Note 17.

NOTES

1. The resolution of A-1 involved a comprehensive set of guidelines and criteria to evalute and upgrade utility training programs per TMI Task Action Plan Item I.A.2.3. This issue has been closed by the accreditation of training programs at the Authority's nuclear power plants, Indian Point 3 and James A. FitzPatrick. Notification of accreditation was received by letter dated 11/4/86 from the National Academy for Nuclear Training.

2. An NRC Safety Evaluation for Indian Point 3 was issued on 3/10/86.

3. The Authority provided its response to Generic Letter 85-02 for Indian Point 3 by letter dated 6/18/86 (IPN-85-031).

4. An NRC Safety Evaluation for Indian Point 3 was issued on 2/21/89. AMSAC installation was completed during the cycle 6/7 refueling outage which ended 6/89.

5. NRC Generic Letter 82-26, dated 11/12/82, issued NUREG-0744 Revision 1. The NUREG specifies that no action is required unless the Charpy V-notch (C_v) impact test upper shelf energy (USE) for Reactor Pressure Vessel beltline materials is less than 50 ft-lb. This issue is monitored by the Reactor Vessel Surveillance Program. All IP3 surveillance capsules that have been analyzed to date have met the 50 ft-lb requirement of 10 CFR 50.60, Appendix H.

6. The NRC informed the Authority by letter dated 12/2/83 that, based on the conclusions of NUREG-0577, USI A-12 was considered resolved for Indian Point 3.

7. This USI will be considered as part of the Indian Point 3 Individual Plant Evaluation which is expected to be provided to the NRC on 9/1/92 (NYPA letter IPN-89-066 dated 10/27/89).

8. An NRC Safety Evaluation for Indian Point 3 was issued on 6/5/85. The NRC concluded that the Equipment Qualification Program for Indian Point 3 was in compliance with the requirements of 10 CFR 50.49.

   As a result of environmental qualification inspections conducted by the NRC, the Authority submitted a report (letter dated 5/8/89) in response to concerns involving cable submergence.

9. The Authority provided its response to Generic Letter 88-11 for IP3 by letter dated 1/13/89. The effects of revision 2 to Regulatory Guide 1.99 are being evaluated. Technical Specifications and procedure changes will be completed prior to startup (scheduled for 10/90) from the cycle 7/8 refueling outage.

10. An NRC Safety Evaluation for Indian Point 3 was issued on 2/13/85 which concluded that the guidelines of NUREG-0612, Sections 5.1.1 and 5.3 were satisfactory and that Phase I of this issue for IP3 was acceptable.

    Phase II of this issue was closed on a generic basis by NRC Generic Letter 85-11, dated 6/28/85.

11. Resolution of this item has been included under the scope of USI A-46. USI A-46 is being resolved on a generic basis by the Seismic Qualification Utility Group (SQUG).

12. The issue was closed on a generic basis as a result of Generic Letter 85-22 dated 12/3/85. The Generic Letter did not require licensee actions, but recommended that Regulatory Guide 1.82, Revision 1 be used as guidance for the conduct of 10 CFR 50.59 reviews dealing with the changeout and/or modification of thermal insulation installed on primary coolant system piping and components. Accordingly, an 10 CFR 50.59 evaluation (NSE 88-03-013 SG) was performed for the IP3 Steam Generator Replacement Project.

13. The Authority provided its response to this issue for IP3 by letter dated 4/17/89. A schedule for implementing modifications and procedure changes will be provided within 30 days of NRC notification per 10 CFR 50.63 (c)(3).

14. This USI will be considered as part of the Indian Point 3 Individual Plant Evaluation which is expected to be provided to the NRC on 9/1/92 (NYPA letter IPN-89-066 dated 10/27/89).

15. This item is being resolved on a generic basis by the Seismic Qualification Utility Group (SQUG). Implementation schedule is based on completion of final SQUG Generic Implementation Procedure (GIP) and issuance of SER by NRC.

16. The expected response date, per Generic Letter 89-19, is 3/19/90.

17. Authority letter dated 1/23/86 provided the response to this issue for IP3. 10 CFR 50.61 requires continuous monitoring of this issue.