

Indian Point 3
Nuclear Power Plant
P.O. Box 215
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Janaury 15, 1988
IP3-WAJ-87-070Z
IP3-87-JJA4-263H

Director of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attention: Document Control Desk

Subject: Indian Point No. 3 Nuclear Power Plant
Docket No. 50-286
Reactor Containment Building
Integrated Leak Rate Test

Dear Sir:

In accordance with the Code of Federal Regulations, Title 10, Part 50, Appendix J, the New York Power Authority provides herewith, five (5) copies of the Reactor Containment Building Integrated Leak Rate Test (ILRT). Also attached, please find a summary of the type "B" and "C" test results since the completion of the last ILRT.

Containment pressurization commenced on July 24, 1987. At 2100 hours on July 25, 1987 the leak rate calculations showed the leak rate to be approximately .3%/day. A leak survey revealed a leak through the Reactor Coolant Pump Seal Water Return Valve, MOV-222 at penetration R, line 17. A manual valve 221A was closed to isolate the leakage from MOV-222. Since MOV-222 is a containment leakage path in accordance with the FSAR and Technical Specifications, the ILRT was restarted. After the completion of the ILRT, MOV-222 was repaired and retested locally. The final local leakage was added to the "As Left" ILRT leakage, and the results were satisfactory.

The final containment "As Left" leakage rates were:

Simple Leak Rate = .038%/Day
Fitted Leak Rate = .029%/Day
95% Upper Confidence Level (UCL) = .050%/Day
Acceptance Criteria (75%La) = .075%/Day

Therefore, the "As Left" leakage rate was determined to be acceptable. Please note that corrections to the ILRT Leakage rate in accordance with NRC I&E Information Notice 85-71 are found in appendix A of the Final Report.

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January 15, 1988
Page 2
Integrated Leak Rate Test

The next required Integrated Leak Rate Test for Indian Point 3 Nuclear Power Plant is tentatively scheduled for the cycle 7/8 refueling outage pending NRC review and approval in accordance with 10CFR50 Appendix J, Paragraph III.A.6(a). This refueling outage is tentatively scheduled for the last quarter of 1990.

Should you or your staff have any questions, please contact Mr. M.P. Cass of my staff.

Very truly yours,



W.A. Josiger
Resident Manager
Indian Point 3 Nuclear Power Plant

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Attachments

ATTACHMENT

REACTOR CONTAINMENT BUILDING INTEGRATED LEAK RATE TEST
Local Leakage Rate (Type B and C) Testing

The combined gaseous leakage* ("B" & "C" type testing) is summarized below for tests performed since the July 1982 Integrated Leak Rate Test.

<u>DATE</u>	<u>TYPE "B"</u>	<u>TYPE "C"</u>	<u>TOTAL</u>	<u>ACCEPTABLE .6 La</u>
MAY 1983	.94 SCFM	.12 SCFM	1.06 SCFM	4.11 SCFM
JAN 1984	.85 SCFM	.05 SCFM	.90 SCFM	4.11 SCFM
SEPT 1985	.54 SCFM	.08 SCFM	.62 SCFM	4.11 SCFM
JULY 1987	.61 SCFM	.11 SCFM	.72 SCFM	4.11 SCFM

* As Found, maximum pathway leakage

The combined liquid leakage rates of the five Isolation Valve Seal Water System zones are listed below for tests performed since the July 1982 Integrated Leak Rate Test.

<u>DATE</u>	<u>TOTAL</u>	<u>ACCEPTABLE PER TECHNICAL SPECIFICATION</u>
MAY 1983	4,776 SCC/HR	14,700 SCC/HR
JAN 1984	9,228 SCC/HR	14,700 SCC/HR
SEPT 1985	1,498.2 SCC/HR	14,700 SCC/HR
JULY 1987	7,156.2 SCC/HR	14,700 SCC/HR